

# Prediction of subcooled flow boiling pressure drops in small circular tubes

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## Abstract

Two-phase pressure drops in a mini-tube, in transition flow and subcooled boiling, are analyzed in the present paper, with the support of an experimental data set provided by ENEA with their BO.E.MI.A. test section. The methodology can be applied to different fluids according to similarity criteria. Single phase, subcooled and saturated conditions have been analysed. The Reynolds number in the experiments was mainly in the transition zone between laminar and turbulent conditions, therefore a third order interpolation curve of the friction factor has been employed. The methodology is based on the model from Delhaye. The model considers the fluid properties, the energy, mass and momentum conservation equations to predict the ONB and OSV points and a hyperbolic function has been adopted to calculate the non-equilibrium vapor quality in the subcooled boiling region. The best agreement with the ENEA experimental data has been obtained using in the methodology the Chisholm correlation, with 83.59% of the predicted values with an error lower than 30%.

*Keywords: Subcooled flow boiling, pressure drops, transition flow*

## 1. INTRODUCTION

Extreme engineering applications often require small high-performing solutions. For heating issues, micro-heat exchangers are widely used, as, for example, in the electronic chips cooling where high heat fluxes must be discharged over a small area. One of the simplest arrangements that can be used for the heat removal involves single-phase forced convection that is, however, limited in terms of efficiency. The higher heat transfer coefficients could be achieved only employing a phase transition, and subcooled flow boiling can be considered the best solution for small-scale heat removal equipment.

Due to the small hydraulic diameter used in mini- and micro-exchangers, excessive pressure drop is always a concern, since these devices are typically used in combination with pumps with limited

32 pumping power capability. Another concern is the pressure oscillation due to hydrodynamic  
33 instabilities that can appear in subcooled boundaries, You et al. [1], and that can lead up to CHF,  
34 Caira et al. [2]. Thus, instabilities must be predicted and prevented to ensure safe operation and good  
35 cooling performance. A few published studies discuss pressure drop and hydrodynamic instability of  
36 flow boiling in mini/micro-tubes. These concerns are compounded when the fluid flow is in transition  
37 between laminar and turbulent flows, where there is no valid and established model.

38 Jacobi et al. [3] proposed a classification based on the physical size of the channels: micro-channels  
39 for a size range 1  $\mu\text{m}$  - 100  $\mu\text{m}$ , meso-channels for channel sizes from 100  $\mu\text{m}$  to 1.0 mm, compact  
40 channels from 1.0 mm to 6.0 mm and, macro-channels for all channel sizes exceeding 6.0 mm.  
41 Instead, Kandlikar et al. [4] proposed a classification based on flow considerations: conventional  
42 channels for hydraulic diameters of 3.0 mm or larger, mini-channels for hydraulic diameters of 200  
43  $\mu\text{m}$  to 3.0 mm, micro-channels for hydraulic diameters smaller than 200  $\mu\text{m}$ . The recommendations  
44 are valid for both single-phase and two-phase systems. Cheng and Wu [5] proposed criteria based on  
45 the Bond number  $Bd = g(\rho_f - \rho_g)D^2/\sigma$ , to consider the properties of the fluid and, therefore, the  
46 gravity and surface tension effects: micro-channel, if  $Bd < 0.05$  (significant effect of surface tension);  
47 mini-channel, if  $0.05 < Bd < 3.0$  (both gravity and surface tension are important); macro-channel, if  
48  $Bd > 3.0$  (surface tension has negligible effect).

49 Pressure drops in saturated flow boiling were largely analyzed at macro-scale, Ould Didi et al. [6]  
50 compared seven of the most quoted macro-scale methods in the literature to determinate frictional  
51 pressure drop on a 788 points database in two horizontal macro-scale test sections of 10.92 and 12.00  
52 mm diameter for five fluorocarbon refrigerants. They found that the methods of Muller-Steinhagen  
53 and Heck [7] and Gronnerud [8] gave the best predictions. Ribatski et al. [9] compared twelve  
54 prediction methods and found as the most effective the macro-scale method proposed in [7].  
55 However, they showed how none of the analyzed methods can be classified as a design tool for  
56 microscale tubes.

57 In micro-scale Zhang and Webb [10], Kuwahara et al. [11] obtained good predictions of their data  
58 for R134a by using the Friedel [12] correlation. Also, Lazarek and Black [13] studied the problem  
59 obtaining good forecasts by using a value of  $C = 30$  in the generalized Chisholm [14] and Lockhart–  
60 Martinelli [15] correlations. Along this direction, Qu and Mudawar [16], Lee and Mudawar [17] and  
61 Lee and Garimella [18] developed flow boiling pressure drop models based on their experimental  
62 data developed in microchannel heat sinks. Mishima and Hibiki [19] obtained reasonably good  
63 predictions for their frictional pressure drop data by correlating the Chisholm [14] parameter in the  
64 Lockhart–Martinelli [15] correlation as a function of the tube diameter. Bowers and Mudawar [20];

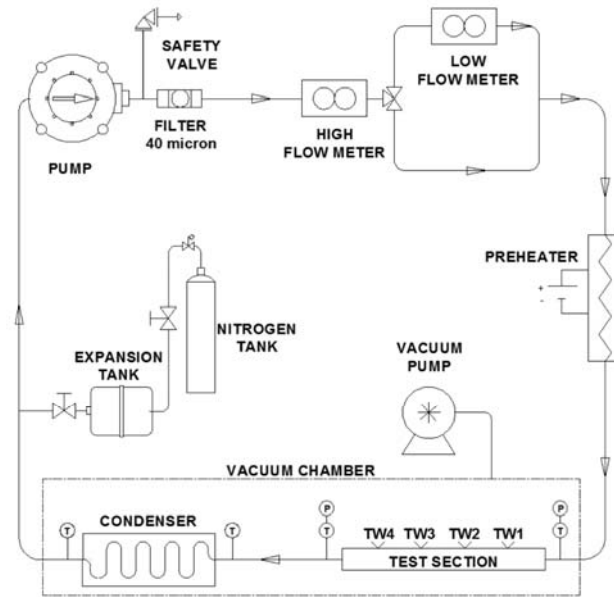
65 21] analyzed flow boiling pressure drop of refrigerant R-113, using a homogenous equilibrium model,  
66 in both mini and micro-channel obtaining a good agreement. Two-phase hydrodynamic instabilities  
67 in parallel mini/micro-channels were addressed by Kandlikar et al. [22] and Hetsroni et al. [23]. Tran  
68 et al. [24] studied flow boiling pressure drop of three different refrigerants in single tubes and for a  
69 single rectangular channel. Kim and Mudawar [25] developed a model, using a database of 2378  
70 experimental points, that takes into account six dimensionless parameters to calculate the Lockhart–  
71 Martinelli  $C$  parameter.

72 Both macro-scale and micro-scale correlations have been used in the literature to develop models for  
73 heat transfer and pressure drops in mini-channels, and mainly in laminar or turbulent conditions. The  
74 pressure drops in a mini-channel in single and two-phase transition flow are analyzed in the present  
75 paper, with the support of experimental data provided by ENEA in their BO.E.MI.A. test section [26].  
76 The main aim of the work is to provide a comprehensive methodology to predict pressure drops for  
77 small circular tubes, in transition flow and subcooled boiling condition, valid for many fluids  
78 according to similarity criteria, due to the lack of specific models in these conditions.

79

## 80 **2. THE BO.E.MI.A. EXPERIMENTAL FACILITY**

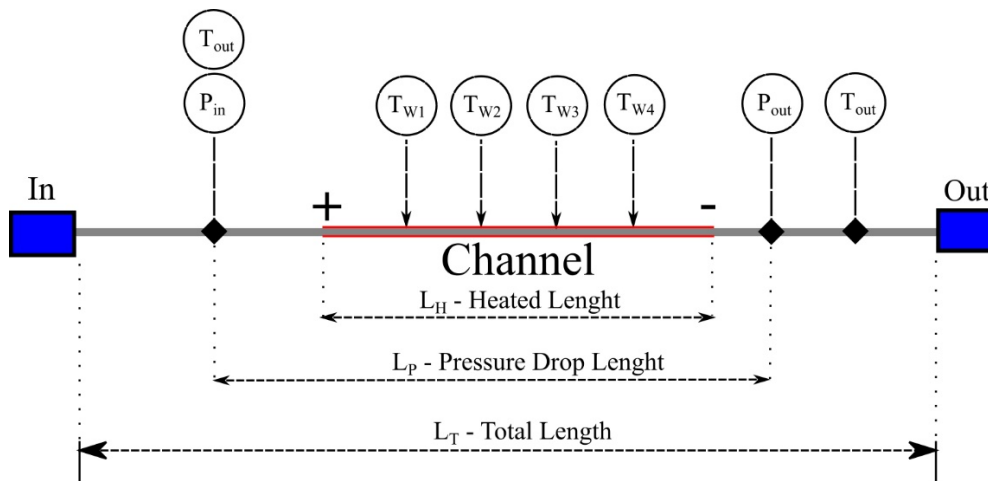
81 The BO.E.MI.A. experimental facility (BOiling Experiments in Microchannel Apparatus) was built  
82 at the ENEA Laboratory of Chemical and Thermo-Fluid Dynamic Processes for Energy. It consists  
83 of a tube of 1.016 mm (internal diameter) and wall thickness 0.57 mm; two different total lengths  
84 have been used, 100 or 200 mm: The working fluid is the refrigerant FC-72 (perfluorohexane C<sub>6</sub>F<sub>14</sub>).  
85 The facility can operate at pressures up to 10 bar and a volumetric flowrate from 6 to 552 ml/min. An  
86 upstream electrical preheater allows the inlet temperature setting up to saturation condition or to a  
87 planned subcooled degree. A counter-current tube-in-tube condenser, cooled by demineralized water,  
88 is placed downstream the section. Fig. 1 shows the facility scheme with the main components:



**Fig. 1** – BO.E.M.I.A. facility simplified layout

89  
90

91 The test rig includes: pressure taps, thermocouples for bulk fluid temperature measurement, power  
 92 supply and four wall thermocouples.  $L_H$  is the heated length, equal to 84 mm for the 200 mm length  
 93 tube and 60 mm for the 100 mm tube.  $L_p$  is the distance between the pressure sensors, equal to 96 mm  
 94 and 70 mm for the 200 mm tube and the 100 mm tube, respectively. A constant power DC supply is  
 95 used to heat uniformly the test section by Joule effect; the maximum heat flux generable along the  
 96 channel is  $150 \text{ kW/m}^2$ . Axial conduction effects are negligible, as discussed in [26]. Fig. 2  
 97 schematically represent the layout of the heated channel in the facility.



**Fig. 2** - Test section layout

98  
99

100 The instruments' uncertainties and the most significant calculated ones are presented in Tab. 1. The  
 101 channel is horizontally oriented. Any further detail can be found in [26] and [27].

102

**Tab. 1** - *Measurement uncertainties*

<i>Mass flow rate (high)</i>	0.15	% of Readings
<i>Mass flow rate (low)</i>	1	% on F.S.
<i>Diameter</i>	25	μm
<i>Temperature</i>	0.40	°C
<i>Pressure</i>	0.08	% on F.S.
<i>Differential pressure</i>	0.075	% of Calibrated Span
<i>Electrical Power</i>	0.48-1.42	% of Readings

103

104 Experimental tests were performed at a pressure of 3 and 4 bar, mass flux is in the range 415-1439  
 105 Kg/m<sup>2</sup>s and 1.5-181 kW/m<sup>2</sup> was the range of the applied heat flux. A total of 161 total pressure drop  
 106 data points (38 in single phase only, 63 up to subcooled flow boiling and 60 reaching saturated boiling  
 107 condition) were collected for the 200-mm tube and 141 data for the 100-mm tube (41 in single phase,  
 108 76 with subcooled flow boiling and 24 up to saturated boiling), for a total of 302 data points, 79 of  
 109 which in single phase condition, 139 up to subcooled flow boiling and 84 including saturated  
 110 conditions also. Inlet subcooling was between 8.3 and 32.4 °C. Reynolds number was from 2500 to  
 111 4500 for the 200-mm tube and between 2750 and 3600 for the 100 mm tube: thus the flow is always  
 112 in transition conditions. The identification of single-phase, subcooled and saturated boiling points is  
 113 based on the calculation of the “onset of nucleate boiling” point (described in Sect. 4) and the energy  
 114 balance to evaluate the bulk saturation point and the saturation length. The test matrixes are shown in  
 115 Tables 2 and 3.

116 Two pressure transducers (0-25 bar) on either side of the mini-tube provided the total pressure drop  
 117 over the channel; a differential manometer (0-6.895 bar) was mounted in parallel to the transducers  
 118 for extra precise differential pressure measurements, as reported in Saraceno et al. [26].

119 According to both Kandlikar et al. [4] criterion (diameter is 1 mm) the experimental tube can be  
 120 classified as a mini-channel. If the and Cheng and Wu [5] criterion is used, the Bond number is  
 121 between 2.7 and 3.3, close to the upper boundary between mini-channel and macro-channel.

122

123

**Tab. 2** - Test matrix for 200 mm tube

<i>Tests series</i>	<i>T</i>	<i>P</i>	<i>Mass flux</i>	<i>q" min</i>	<i>q" max</i>	<i>Total points</i>	<i>Single phase</i>	<i>Up to Subcooled boiling</i>	<i>Up to Saturated boiling</i>	<i>ΔT<sub>sub,in</sub></i>	<i>Re<sub>in</sub></i>
-	°C	Bar	kg/m <sup>2</sup> s	W/m <sup>2</sup>	W/m <sup>2</sup>	n°	n°	n°	n°	°C	-
<b>1</b>	84	3.08	1126	4357	98414	32	3	14	15	13.78	3250
<b>2</b>	88	5.02	1234	4227	181562	32	19	11	2	32.42	3475
<b>3</b>	87	4.08	1234	4297	170409	18	8	5	5	24.22	3450
<b>4</b>	97	4.04	1030	3297	97900	13	1	5	7	9.12	3450
<b>5</b>	97	4.05	925	3188	99653	15	1	5	9	9.22	3200
<b>6</b>	98	4.04	824	3056	86153	17	2	5	10	9.02	2880
<b>7</b>	84	3.05	1235	4143	71069	15	2	10	3	13.21	2500
<b>8</b>	97	4.05	1439	7406	87896	8	1	3	4	9.06	4500
<b>9</b>	87	3.04	1132	4070	70838	11	1	5	5	8.32	3300
						Total	161	38	63	60	

124

125

**Tab. 3** - Test matrix for 100 mm tube

<i>Tests series</i>	<i>T</i>	<i>P</i>	<i>Mass flux</i>	<i>q" min</i>	<i>q" max</i>	<i>Total points</i>	<i>Single phase</i>	<i>Up to Subcooled boiling</i>	<i>Up to Saturated boiling</i>	<i>ΔT<sub>sub,in</sub></i>	<i>Re<sub>in</sub></i>
-	°C	Bar	Kg/m <sup>2</sup> s	W/m <sup>2</sup>	W/m <sup>2</sup>	n°	n°	n°	n°	°C	-
<b>1</b>	65	3.04	1149	7685	76510	16	13	3	0	32.28	2750
<b>2</b>	78	3.02	1131	7687	76510	16	6	10	0	19.74	3050
<b>3</b>	85	3.07	1148	2501	47315	10	3	7	0	11.00	3350
<b>4</b>	86	3.08	1139	7422	80253	17	3	12	2	13.21	3250
<b>5</b>	86	3.09	1126	8718	127005	21	2	11	8	11.75	3250
<b>6</b>	85	3.04	1032	1612	113401	25	3	11	11	12.60	2950
<b>7</b>	91	4.03	1223	2923	116891	24	7	14	3	17.89	3600
<b>8</b>	93	4.01	1138	2286	57113	12	4	8	0	12.96	3500
						Total	141	41	76	24	

126

127

128

129 **3. SIMILARITY CRITERIA**

130 An important issue in utilizing the available correlations is their applicability when different fluids  
 131 are used. To overcome these difficulty, Delhaye et al. [28] developed some criteria based on the Kay  
 132 and Nedderman [29] assumptions on enthalpy. These criteria may be applied both to models and  
 133 correlations and they are useful to understand if the specific correlation is applicable regardless the  
 134 fluid and the proprieties adopted. The similarity criteria can be summarized in the following  
 135 requirements:

- 136     ▪ Same channel shape: different shapes influence the boundary layers and the heat flux  
 137         distribution along the channel.
- 138     ▪ Same vapor to liquid density ratio at the respective pressures:

$$139 \quad \left(\frac{\rho_v}{\rho_l}\right)_{p(fluid(a))} = \left(\frac{\rho_v}{\rho_l}\right)_{p(fluid(b))} \quad (1)$$

140 To avoid any difference in the volume occupied by every phase the ratio between the specific  
 141 volume of the two phases should be the same at the saturation pressure.

- 142     ▪ Same Weber number, to derive the equivalent mass flux  $G$ :

$$143 \quad We \cong \frac{G^2 D}{\sigma \rho_{l,sat}} \Rightarrow G_{fluid(b)} = \sqrt{\frac{We_{fluid(a)} \sigma \rho_{l,sat}}{D}} \quad (2)$$

144 The inertia/surface tension ratio should be the same to grant the same film, droplet or bubbles  
 145 dimension.

- 146     ▪ Same Boiling Number, to calculate the equivalent heat flux  $Q$ :

$$147 \quad Bo \cong \frac{Q}{GH_{lv}} \Rightarrow Q_{fluid(b)} = \frac{H_{lv,fluid(b)} \cdot Q_{fluid(a)} \cdot G_{fluid(b)}}{H_{lv,fluid(a)} \cdot G_{fluid(a)}} \quad (3)$$

148 The Boiling number can be thought of as the ratio of the produced vapor mass flux to the total  
 149 mass flux. A different mass of vapor generated can change the vapor distribution in the  
 150 channel up to the thermal crisis.

- 151     ▪ Same equilibrium quality, to calculate the equivalent inlet temperature:

$$152 \quad x_{eq,in} \cong \frac{H_{l,in} - H_{l,sat}}{H_{lv}} \Rightarrow$$

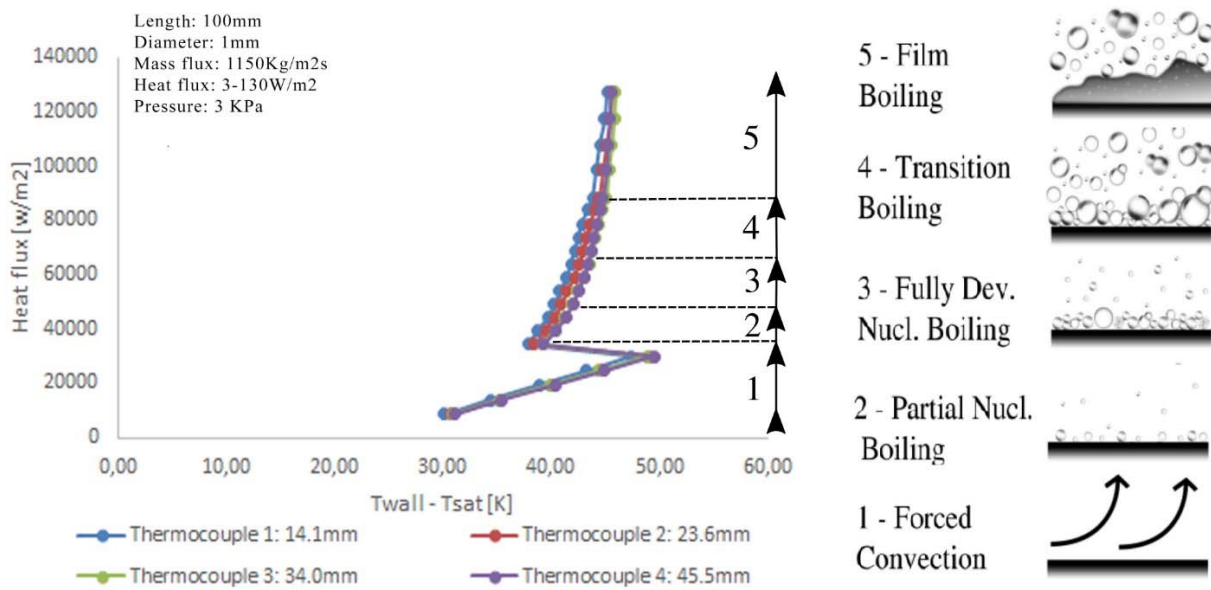
$$153 \quad H(T)_{l,in fluid(b)} = H_{l,sat fluid(b)} + \frac{H_{lv,fluid(b)}}{H_{lv,fluid(a)}} (H_{l,in} - H_{l,sat})_{fluid(a)} \quad (4)$$

154 The use of the same equilibrium quality to calculate an equivalent inlet temperature is useful  
 155 to grant the same energetic inlet condition for the fluid.

156

157 Wetting fluids, as FC-72, show a high thermal hysteresis in high subcooled condition at the inlet, see  
 158 Fig. 3, as reported in the work of Celata et al. [30].

159 Due the limited literature concerning the use of FC-72, the similarity criteria have been largely  
 160 adopted in this paper, to understand if a correlation is applicable to the BO.E.M.I.A. experimental  
 161 setup.



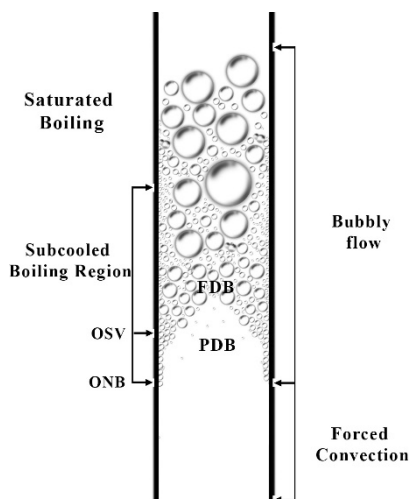
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163 **Fig. 3** - Heat flux vs wall superheating at 1150 Kg/m<sup>2</sup>s on 100 mm tube, boiling phases

164

#### 165 4. SUBCOOLED BOUNDARIES CALCULATION

166 Single phase forced convection ends when the first vapor bubbles appear at the first nucleation site at  
 167 the ONB (Fig. 4). When a significant increase in the void fraction occurs, the fluid reaches the “onset  
 168 of significant void” (OSV). Finally, saturated boiling starts when the whole mixture is in saturated  
 169 conditions. Once defined ONB and OSV, it is possible to distinguish two regions in the subcooled  
 170 zone: the partially developed boiling (PDB) region, delimited by the ONB and the OSV, and the fully  
 171 developed boiling (FDB) region which is delimited by the OSV and the saturation point.

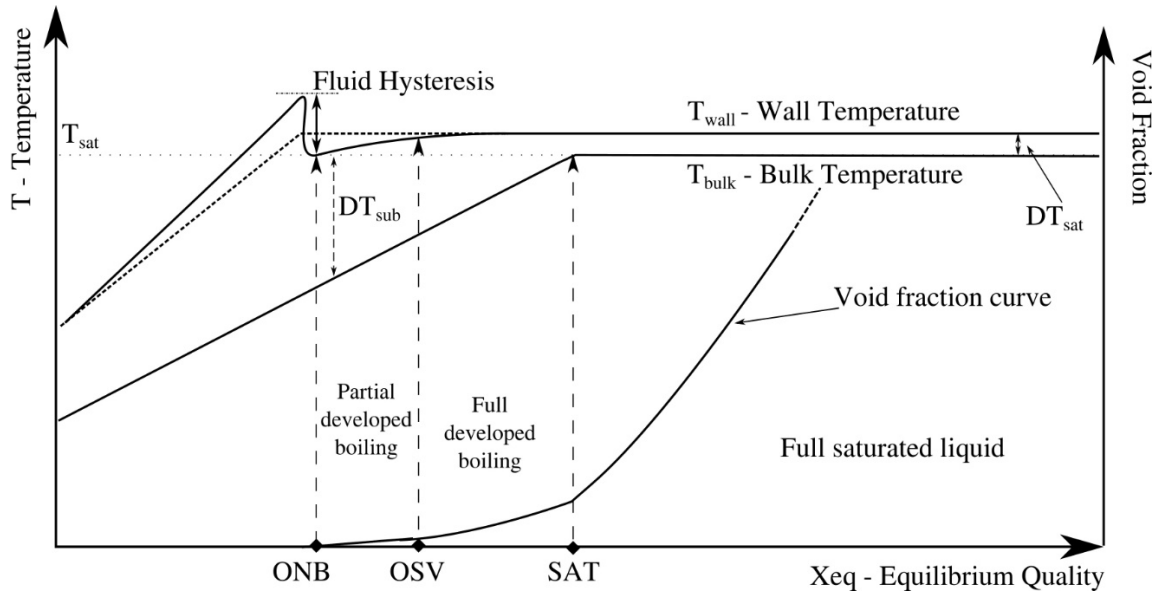


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**Fig. 4** - Subcooled flow boiling representation in a vertical channel



173 The void fraction increases slightly from the ONB to the OSV and it increases much faster in the  
 174 FDB region. The wall temperature increases linearly in single phase flow and remains almost constant  
 175 in the boiling region, as shown in Fig. 5.



176  
 177 **Fig. 5** - Wall temperature, bulk temperature and void fraction trends at the increasing of equilibrium quality  
 178

179 The Onset of Nucleate Boiling (ONB) can be identified by an energy balance, as stated in [28]:

$$180 \quad Z_{ONB} = \frac{Gc_{pl}D}{4} \cdot \left[ \frac{((T_{sat} - T_{l,in}) + (\Delta T_{sat})_{ONB})}{Q} - \frac{1}{h_{l,conv}} \right] \quad (5)$$

181 The convective heat transfer coefficient  $h_{l,conv}$  can be calculated by the well-known Petukhov-  
 182 Gnielinski correlation, where the fluid properties are evaluated at the film temperature:

$$183 \quad \frac{(T_l(z_{ONB}) + T_{wall}(z_{ONB}))}{2} \quad (6)$$

184 The only exception is the liquid heat capacity, that is calculated at the average temperature between  
 185 the inlet and the ONB:

$$186 \quad \frac{(T_l(z_{ONB}) + T_{l,in})}{2} \quad (7)$$

187 The wall superheating ( $\Delta T_{sat} = T_w - T_{sat}$ ) at the ONB point can be calculated by a modified Frost  
 188 and Dzakowic [31] correlation for water:

$$189 \quad (\Delta T_{sat})_{ONB} = \left( \frac{8\sigma Q T_{sat}}{k_{l,sat} H_{lv,sat} \rho_g} \right)^{0.5} Pr_l^{0.95} \quad (8)$$

190 The  $Pr$  number exponent has been changed in this work from 1 to 0.95, to better agree with the  
 191 available experimental results. All the proprieties in Eq. (8) are calculated at the saturation  
 192 temperature  $T_{sat}$ . Tab. 4 shows the equivalent applicability range for FC-72 using similarity criterion,  
 193 as described in Eqs. (1) to (4):

194 **Tab. 4** - Frost and Dzakowic correlation [31]: applicability range for FC-72

Parameters	Water	FC-72	BO.E.MI.A. data
Pressures	0.1-20 MPa	0.0075-1.57 MPa	0.3-0.5 MPa
Mass velocity	No restriction	No restriction	415-1439 kg/m <sup>2</sup> s
Heat flux	150 kW/m <sup>2</sup>	6.5 kW/m <sup>2</sup>	1.5-181 kW/m <sup>2</sup>

195 The liquid temperature at the ONB point, to be used in the fluid proprieties calculation by Eqs. (6)  
 196 and (7), is calculated iteratively from the energy balance, as in Eq. (9):

$$197 \quad T(x) = T_{in} + 4 \frac{Q \cdot Z_{ONB}}{G \cdot c_{pl} \cdot D} \quad (9)$$

198 The heat capacity  $c_{pl}$  is calculated at the average temperature between  $T_{in}$  and  $T(x)$  where  $x$  is  $Z_{ONB}$ .

199 The Onset of Significant Void point is identified through the Saha and Zuber correlation [32], as  
 200 suggested in the Delhaye et al. model [28]. It is calculated starting from the ONB point. The liquid  
 201 bulk temperature at the OSV is:

$$202 \quad T_{l,OSV} = \Delta T_{OSV} - T_{sat} \quad (10)$$

203 where  $\Delta T_{OSV}$  is the subcooling degree and it is calculated according to the Saha and Zuber [32] criteria  
 204 to determine the bubble departure point:

$$205 \quad \blacksquare \quad Pe < 700000; \quad \Delta T_{OSV} = 0.0022 \cdot \frac{QD}{k_l} \quad (11a)$$

$$206 \quad \blacksquare \quad Pe > 700000; \quad \Delta T_{OSV} = 153.8 \cdot \frac{Q}{G \cdot c_{p,l}} \quad (11b)$$

207 where:

$$208 \quad Pe = \frac{G c_{p,l} D}{k_l} \quad (12)$$

209 The fluid properties are calculated at the OSV temperature. From the thermal balance at the OSV  
 210 point, it is possible to identify the axial position ( $Z_{osv}$ ) where OSV starts:

$$211 \quad 212 \quad Z_{OSV} = Z_{ONB} + [H_l(T_{l,OSV}) - H_l(T_{l,ONB})] \cdot \frac{GD}{4Q} \quad (13)$$

213 ONB and OSV points will be used as boundaries in the void fraction calculations for the partial and  
214 fully developed boiling regions.

215

## 216 5. VOID FRACTION CALCULATION

217 Two-phase pressure drop calculation needs the evaluation of the void fraction. The Zuber and Findlay  
218 drift flux model [33] is widely used in the literature to evaluate the void fraction  $\alpha$ :

$$219 \quad \alpha = \frac{\Gamma_v}{C_0 \Gamma + V_g} \quad (14)$$

220 where  $C_0$  is the distribution parameter, that is a function of the local vapor void fraction and the local  
221 mixture velocity;  $V_g$  is the weighted drift velocity, that physically depends on the radial profile of the  
222 void fraction and can be calculated as a function of local the vapor void fraction and the local vapor  
223 velocity;  $\Gamma$  is the volumetric flow rate. These parameters, in the original work, are semi-empirical  
224 and based on a fluid database. Lahey and Moody [34] proposed different methods to calculate the  
225 void fraction in subcooled boiling, based on the Zuber and Findlay [33] model, that was developed  
226 to estimate the void fraction in the fully developed boiling region.

$$227 \quad \alpha = \frac{x_v \cdot \rho_l \cdot G}{C_0 (x_v \cdot \rho_l + (1 - x_v) \cdot \rho_v) G + V_g \cdot \rho_l \cdot \rho_v} \quad (15)$$

228 The differences of the Lahey and Moody [36] model from the original one are: (a) the relation  
229 between actual vapor quality and equilibrium quality, (b) the distribution parameter  $C_0$  calculation  
230 and (c) the weighted drift velocity  $V_g$  calculation. Most of the available models calculate the void  
231 fraction in fully developed boiling region assuming a zero quality in the partially developed boiling  
232 region. However, Levy [35] and Griffith et al. [36] proposed correlations for the void fraction at the  
233 OSV. Delhaye et al. [28] improved the Lahey and Moody [34] extending the range up to the end of  
234 subcooled region, concatenating different approaches to different regions of subcooled boiling.

### 235 (a) Relation between non-equilibrium and equilibrium vapor quality in the subcooled region

236 The vapor quality  $x_v$  is the vapor mass fraction in a mixture. It is different but related with the  
237 equilibrium quality  $x_{eq}$ :

$$238 \quad x_{eq}(z) = \frac{H(z) - H_{l,sat}}{H_{lv}} \quad (16)$$

239 The original model from Lahey and Moody [34] assumes the quality between ONB and OSV (PDB  
240 region) equal to zero. However, this approach would lead to an overestimation of the void fraction  
241 between the OSV and the saturation point (FDB zone). Therefore, a modified non-equilibrium quality

242 model which adopts an approximation of the subcooled void in PDB and FD regions should be used.  
 243 Delhaye et al. [28] proposed a hyperbolic tangent to approximate the transition.

$$244 \quad x_v(Z) = 0.01\xi \left\{ x_{eq}(Z) - x_{eq}(Z_{ONB}) \left[ \tanh \left( \left( \frac{x_{eq}(Z)}{x_{eq}(Z_{ONB})} \right) - 1 \right) + 1 \right] \right\} \quad (17)$$

$$245 \quad x_v(Z) = x_{eq}(Z) \text{ if } x_{eq}(Z) \geq x_v(Z)$$

246  $\xi$  is a custom constant that must be identified to allow the first order continuity of the quality function  
 247 between the PDB and FDB regions at the OSV point, identified by Eq. (13), where the vapor quality  
 248 at OSV is expressed by the formula:

$$249 \quad x_{v,OSV} = \frac{1}{\left( \left( \frac{\rho_l}{\rho_v} \right) \cdot \left( \frac{1-\alpha_{OSV}}{\alpha_{OSV}} \right) + 1 \right) \alpha_{OSV}} \quad (18)$$

250 where  $\alpha_{OSV}$  is the void fraction at OSV originally calculated by Griffith et al. [36] for water. Tab. 5  
 251 summarizes the applicability range for the original Griffith et al. [36] model.

252 **Tab. 5** - Griffith [36] OSV model applicability range

Parameter	Water	FC-72	BO.E.M.I.A. data
Pressure	3.4 – 6.9 – 10.3 MPa	0.25 – 0.57 – 0.8 MPa	0.3-0.5 MPa
Mass velocity	80 – 400 kg/m <sup>2</sup> s	155 – 1500 kg/m <sup>2</sup> s	415-1439 kg/m <sup>2</sup> s
Heat flux	1600 – 8500 kW/m <sup>2</sup>	80 – 365 kW/m <sup>2</sup>	1.5-181 kW/m <sup>2</sup>

253

254 To extend the applicability to other fluids Delhaye et al. [28] changed that model, introducing the  
 255 capillarity length:

$$256 \quad \alpha_{OSV} = \frac{4a}{D} \quad (19)$$

257 where:

$$258 \quad a = 7.5 \frac{Qk_l Pr_l}{h_l^2 [T_{sat} - T_l(z_{OSV})]} \frac{L_{cap}}{D} \quad (20)$$

259 The capillary length  $L_{cap}$  is defined as:

$$260 \quad L_{cap} = \sqrt{\frac{\sigma}{g(\rho_l - \rho_v)}} \quad (21)$$

261 The single-phase heat transfer coefficient  $h_l$  is evaluated by the Dittus-Boelter correlation and all the  
 262 fluid properties are calculated by Eq. (9) at the OSV temperature.

263 (b) *Distribution parameter*

264 The distribution parameter  $C_0$  used in Eq. (15) is calculated by the equation from Nabizadeh et al.  
265 [37]:

$$266 \quad C_0 = \left(1 + \frac{1-x_v}{x_v} \cdot \frac{\rho_v}{\rho_l}\right)^{-1} \cdot \left(1 + \frac{1}{n} \cdot Fr^{-0.1} \cdot \left(\frac{\rho_v}{\rho_l}\right)^n \cdot \left(\frac{1-x_v}{x_v}\right)^{\frac{11 \cdot n}{9}}\right) \quad (22)$$

267 where:

$$268 \quad n = \sqrt{0.6 \frac{\rho_l - \rho_v}{\rho_l}} \quad (23)$$

269  $Fr$  is the Froude number defined as:

$$270 \quad Fr = \frac{G^2}{g \cdot D \cdot \rho_l^2} \quad (24)$$

271 The void fraction  $\alpha$  is largely influenced by the distribution parameter; Delhaye et al. [28] defines  
272 Eq. (22) as the most promising in the literature, because it involves pressure, mass flux and quality.  
273 Other equations, as from Saha and Zuber [32], use a constant value and Dix [38] uses a function of  
274 quality. Also in [28] a new correlation was proposed, but it is specific for their facility and not suitable  
275 for FC-72.

276 (c) *Weighted drift velocity*

277 Eq. (14) is weakly influenced by the weighted drift velocity and the original formula proposed by  
278 Zuber and Findlay [33] is accurate enough for the calculation, as stated in [28]:

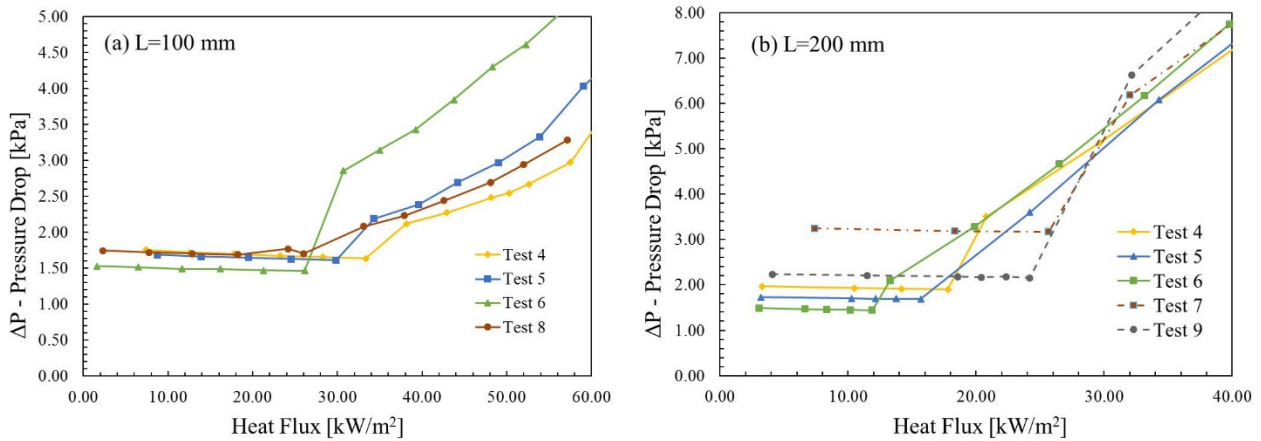
$$279 \quad V_g = 1.41 \left(\frac{\sigma \cdot g(\rho_l - \rho_v)}{\rho_l^2}\right)^{0.25} \quad (25)$$

280

## 281 6. PRESSURE DROPS IN FLOW BOILING

282 Some examples of pressure drop trends at different heat fluxes, from the BO.E.MI.A. experiments,  
283 are shown in Fig. 6. The same trends were also observed from other authors, as in Kim and Mudawar  
284 [39].

285 The pressure drop increment is not constant and four different zones can be identified (Fig. 7).



**Fig. 6** – Total pressure drops at different heat fluxes, for 100mm (a) and 200 mm (b) tubes.

286  
287

288 The subcooled fluid enters the channel (1) and the pressure drops are related only to the liquid  
 289 frictional losses  $(dp/dz)_f$ . When the fluid reaches the subcooled boiling point at ONB the bubbles that  
 290 are formed on the tube reduce the available spaces in the channel accelerating the fluid and  
 291 consequently pressure drop increases (2). However, the embryo bubbles at ONB are formed in  
 292 cavities and when they emerge encounter a large temperature gradient causing bubble reduction or  
 293 implosion, with the resulting instabilities. The instabilities related to the wall superheating hysteresis  
 294 are particularly marked for wetting fluids as fluorocarbons (You et al. [1]). In the full developed  
 295 boiling region (3), before the saturation point, the acceleration pressure drop  $(dp/dz)_{acc}$  quickly  
 296 increases with the void fraction. The frictional losses are higher due to the presence of two phases  
 297 flowing through the tube. In region (2) and (3), the single-phase model fails to predict pressure drops  
 298 and it is necessary to adopt a new model. The void fraction, thanks to its relation to the acceleration  
 299 and frictional pressure drops, can be used as the main parameter to adapt usual two-phase models,  
 300 such as Lockhart-Martinelli [15], Friedel [12], and others. The choice of using two-phase models in  
 301 the subcooled region instead of developing specific correlations is the wide validity range of the  
 302 model and the possibility of use only one model for all the boiling zones. Margulis and Shwageraus  
 303 [40] followed the same approach in their work, using the Osmachkin and Borisov [41] correlation.  
 304 After the saturated point, the pressure drops are well known (4) and the traditional two-phase models  
 305 can be used.

306 The total pressure drop is expressed as:

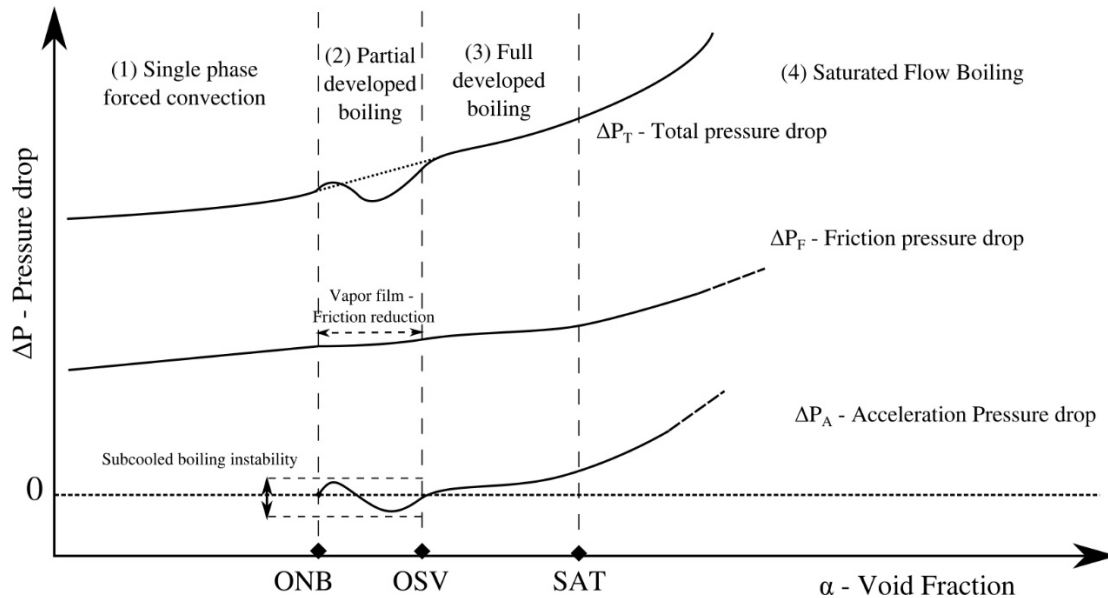
$$307 \quad - \left( \frac{dp}{dz} \right)_t = - \left( \frac{dp}{dz} \right)_f - \left( \frac{dp}{dz} \right)_{acc} - \left( \frac{dp}{dz} \right)_g \quad (26)$$

308 In the single-phase zone, only the friction contribution is considered.

309 The acceleration contribution, also considered in the subcooled region, is here expressed in terms of  
 310 the vapor quality  $x_v$  instead of the equilibrium quality  $x_{eq}$ , as in Eq. (27):

$$311 \quad -\left(\frac{dp}{dz}\right)_{acc} = \frac{G^2}{\rho_l} \left( \frac{\rho_l x_v^2}{\rho_v \alpha} + \left( \frac{(1-x_v)^2}{(1-\alpha)} - 1 \right) \right) \quad (27)$$

312 Fig. 7 shows the emphasized theoretical trends; the gravitational pressure drop  $(dp/dz)_g$  is null because  
 313 the channel is horizontal.



314  
 315 **Fig. 7 – Pressure drop contributions at different void fractions**

316

### 317 **6.1 Pressure drops in single phase transition flow**

318 In the analyzed tests, the fluid is subcooled at the inlet and a zone of the tube is in single-phase forced  
 319 convection until the onset of nucleate boiling occurs. As described, the Reynolds number in the ENEA  
 320 experiments varied from 2750 to 4500. Being these values between the conservative range  $2000 < Re$   
 321  $< 4000$ , the flow is mainly in transition between the laminar and turbulent flow. In this region, there  
 322 are no reliable models able to describe the phenomenon. Several empirical equations have been  
 323 proposed for computing the transitional pressure drop: Brownlie [42]; Cheng and Chiew [43]; Ligrani  
 324 and Moffat [44]; Yalin and Da Silva [45]. However, as shown in [46], a simple interpolation method  
 325 may work better in calculating pressure drop in the transition range.

326 In a first approximation, the pressure drop must vary between the boundaries of laminar and turbulent  
 327 flow. Thus, if the channel is long enough, it is possible to consider the average pressure drop in a  
 328 section of the channel, neglecting the physical oscillation between laminar and turbulent flows. In  
 329 fact, in the same section and over a short time interval, the fluid statistically changes its behavior

330 cyclically. Then, the friction factor variations can be related to the Reynolds number by a polynomial  
 331 interpolation between the laminar and turbulent friction factor. To avoid any discontinuity in the  
 332 functions at the boundaries, a third order polynomial has been adopted. This polynomial function can  
 333 be expressed as:

$$334 \quad f_{tran} = \alpha \cdot Re^3 + \beta \cdot Re^2 + \gamma \cdot Re + \delta \quad (28)$$

335 The 4 constants  $\alpha, \beta, \gamma, \delta$  can be obtained by a 4-equations system, where the following boundary  
 336 conditions are imposed:

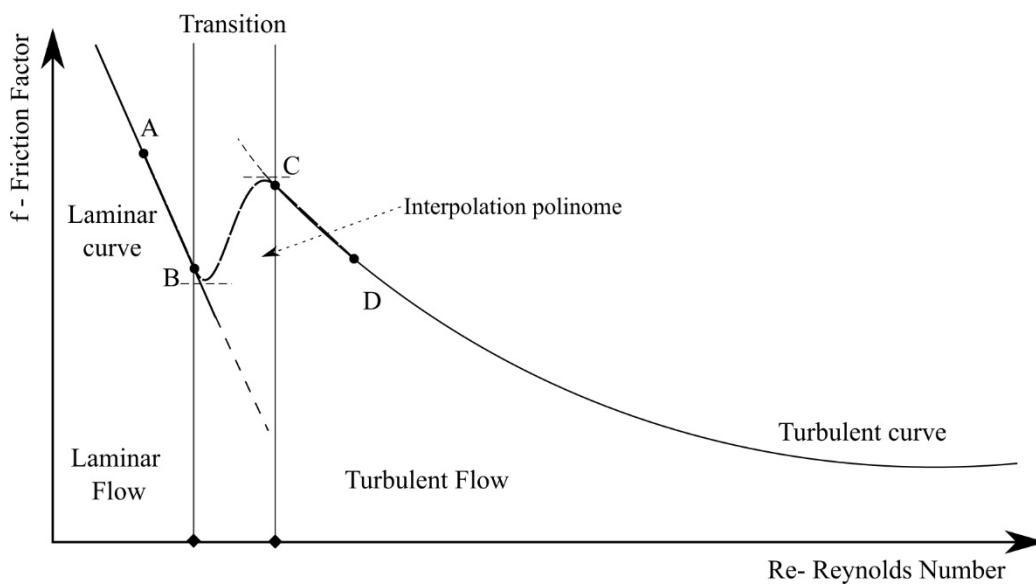
$$337 \quad \begin{cases} A: f_{lam} = f_{tran} & \text{For } Re = 1100 \\ B: f_{r,trans} = f_{turb} & \text{For } Re = 8000 \\ C: f_{lam}' = f_{tran}' & \text{For } Re = 2000 \\ D: f_{tran}' = f_{turb}' & \text{For } Re = 4000 \end{cases} \quad (29)$$

338 The first two conditions are taken far from the boundaries, when the fluid is certainly laminar or  
 339 turbulent; the second two conditions are needed to assure the function continuities at the boundaries.  
 340 The two well know equations used for laminar and turbulent pressure drops in single phase are:

$$341 \quad \text{Darcy friction factor} \quad f_{lam} = \frac{64}{Re} \quad (30)$$

$$342 \quad \text{Colebrook friction factor} \quad 1/\sqrt{f_{turb}} = -2 \log \left( \frac{\varepsilon}{3.7D} + \frac{2.51}{Re\sqrt{f_{turb}}} \right) \quad (31)$$

343 The Colebrook equation (31) [47] is solved recursively.  $\varepsilon$  is absolute roughness of the wall,  $D$  is the  
 344 channel diameter, and  $Re$  is the Reynolds number. Fig. 8 represents schematically the friction factor  
 345 trend.



346  
 347 **Fig. 8** - Interpolation curve for the transition friction factor; A, B, C, D are the 4 boundary condition points.



348 Then, the frictional pressure drop in the subcooled liquid from inlet up to the ONB can be calculated  
 349 by the equation:

$$350 \quad \Delta P_f = f \frac{z_{ONB}}{D} \frac{G^2}{2\rho_l} \quad (32)$$

351  
 352 **6.2 Pressure drops prediction in subcooled and saturated boiling**

353 As discussed before, the experimental channel can be classified as a mini-channel. The two-phases  
 354 models selected to validate the present methodology for the subcooled and saturated boiling zones  
 355 are: Friedel [12], Chisholm [14], Lockhart-Martinelli [15], Chawla [48] and Müller-Steinhagen and  
 356 Heck [49], summarized in Table 7. To grant the wider possible compatibility with different fluids and  
 357 channel diameters, only general models were chosen for the methodology.

358 To compare the methodology with some specific subcooled correlations for mini-channels also, those  
 359 by Owens-Schrock [50] and Tong [51] have been also selected. These correlations have been used to  
 360 calculate the pressure drop in the subcooled flow boiling region only, between ONB and the saturation  
 361 curve. The ONB and OSV points are calculated by the proposed methodology and used to calculate  
 362  $Z_{sub}$  and  $Z_{sat}$  used in the literature correlations. Owens and Schrock's correlation [50] was developed  
 363 for water flow in 3 and 4.6 mm tubes. The correlations of Tong et al. [51], was developed for water  
 364 flow in 1.05–2.44 mm tubes and with two different length-to-diameter ratios. Their applicability  
 365 ranges, as discussed in Sect. 4, are shown in Table 6.

366  
 367 **Tab. 6** Applicability Criteria for Owens and Schrock's [50] and Tong et al. [51] correlations.

<i>Owens and Schrock [50]</i>	<i>Water</i>	<i>FC-72</i>	<i>BO.E.M.I.A. data</i>
<i>Pressure</i>	0.34 – 2.76 MPa	0.02 – 0.20 MPa	0.3-0.5 MPa
<i>Mass velocity</i>	1143 – 5322 kg/m <sup>2</sup> s	745 –3370 kg/m <sup>2</sup> s	415-1439 kg/m <sup>2</sup> s
<i>Heat flux</i>	675 – 4000 kW/m <sup>2</sup>	20 – 108 kW/m <sup>2</sup>	1.5-181 kW/m <sup>2</sup>
<i>Tong et al. [51]</i>			
<i>Pressure</i>	0.4 – 1.6 MPa	0.03 – 0.11 MPa	0.3-0.5 MPa
<i>Mass velocity</i>	25000 – 45000 kg/m <sup>2</sup> s	16220 –28685 kg/m <sup>2</sup> s	415-1439 kg/m <sup>2</sup> s
<i>Heat flux</i>	0 – 80000 kW/m <sup>2</sup>	0 – 217.5 kW/m <sup>2</sup>	1.5-181 kW/m <sup>2</sup>

368

369 The ranges of applicability of the Owens and Schrock correlation are closer than those of Tong  
 370 correlation to the BO.E.MI.A. database.

371 The experimental data are related to the total pressure drops from the inlet to the outlet and the fluid  
 372 enters in the tube in liquid single phase with a high degree of subcooling. The pressure drop data  
 373 where the fluid reaches saturation (54 points) are not considered in the assessment of subcooled  
 374 boiling method, but only in assessing the whole methodology. The experimental setup is provided  
 375 with the pressure transducer at inlet and outlet, thus only the total pressure drop, from inlet to outlet,  
 376 was available for the assessment.

377 The proposed methodology uses a different correlation to calculate pressure drops for each region. In  
 378 single phase Eq (32) was used; for subcooled and saturated flow boiling zones the correlations  
 379 summarized in Tab. 7 were used, with the void fraction and quality obtained from Eqs. 15 and 17,  
 380 respectively. However, it should be noted that the subcooled and saturated boiling correlation cannot  
 381 be the same in the two zones. In fact, the best agreement was obtained using different correlations to  
 382 best fit the data. For comparison purposes, two subcooled boiling correlations (Owens-Schrock [50],  
 383 and Tong [51]) are reported in Table 8 and all the correlations are assessed in Tables 9-11.

384 **Tab. 7 – Two-phase pressure drop models**

Author(s)	Equations
Lockhart-Martinelli [15] (1.49 < D < 25.4 mm)	$\left(\frac{dp}{dz}\right)_f = \Phi_l^2 \left(\frac{dp}{dz}\right)_l$ $\left(\frac{dp}{dz}\right)_l = 4f_l \frac{1}{D_h} G^2 (1 - x_v)^2 \left(\frac{1}{2\rho_l}\right)$ $\Phi_l^2 = 1 + \frac{C}{X_{tt}} + \frac{1}{X_{tt}^2}$ $X_{tt} = \left(\frac{\mu_l}{\mu_v}\right)^{0.1} \left(\frac{\rho_v}{\rho_l}\right)^{0.5} \left(\frac{1 - x_v}{x_v}\right)^{0.9}$ $C = \begin{cases} 20 & Re_l \geq 4000; Re_v \geq 4000 \\ 10 & Re_l \geq 4000; Re_v \leq 2000 \\ 12 & Re_l \leq 2000; Re_v \geq 4000 \\ 5 & Re_l \leq 2000; Re_v \leq 2000 \\ \text{interpolated} & \text{other ranges} \end{cases}$ $f_l = \frac{0.079}{\sqrt[4]{Re_l}}$ $Re_{lo} = \frac{GD}{\mu_l}; Re_{vo} = \frac{GD}{\mu_v}$
Friedel [12] (D > 4 mm)	$\left(\frac{dp}{dz}\right)_f = \Phi_l^2 \left(\frac{dp}{dz}\right)_l$ $\left(\frac{dp}{dz}\right)_l = 4f_l \frac{1}{D} G^2 (1 - x_v)^2 \left(\frac{1}{2\rho_l}\right)$ $\Phi_l^2 = E + \frac{3.24F \cdot M}{Fr_h^{0.045}} We^{-0.035}$

$$f_l = \frac{0.079}{\sqrt[4]{Re_l}}; \quad f_v = \frac{0.079}{\sqrt[4]{Re_v}}$$

$$Fr = \frac{G^2}{\rho_l^2 \cdot g \cdot D}$$

$$We_{lo} = \frac{G^2 D}{\rho_l \sigma}$$

$$E = (1 - x_v)^2 + x_v^2 \frac{\rho_l f_v}{\rho_v f_l}$$

$$F = x_v^{0.78} (1 - x_v)^{0.224}$$

$$M = \left(\frac{\rho_l}{\rho_v}\right)^{0.91} * \left(\frac{\mu_v}{\mu_l}\right)^{0.19} \left(1 - \frac{\mu_v}{\mu_l}\right)^{0.7}$$

Chawla [48]

(6 < D < 154 mm)

$$\left(\frac{dp}{dz}\right)_f = \Phi_l \left(\frac{dp}{dz}\right)_l$$

$$\left(\frac{dp}{dz}\right)_l = 4f_l \frac{1}{D} G^2 (1 - x_v)^2 \left(\frac{1}{2\rho_l}\right)$$

$$\Phi_l = x^{1.75} \left(1 + S \frac{(1 - x_v) \rho_v}{x_v \rho_l}\right)^{2.375}$$

$$\frac{1}{S} = 9.1 \frac{1 - x_v}{x_v} (Re_v Fr)^{-0.167} \left(\frac{\rho_l}{\rho_v}\right)^{-0.9} \left(\frac{\mu_l}{\mu_v}\right)^{0.5}$$

$$Fr = \frac{G^2}{\rho_l^2 \cdot g \cdot D}$$

Chisholm [14]

(1.49 < D < 25.4 mm)

$$\left(\frac{dp}{dz}\right)_f = \Phi_l^2 \left(\frac{dp}{dz}\right)_l$$

$$\left(\frac{dp}{dz}\right)_l = 4f_l \frac{1}{D} G^2 \left(\frac{1}{2\rho_l}\right)$$

$$f_{l,v} = \begin{cases} \frac{0.079}{\sqrt[4]{Re_{l,v}}} & Re_{l,v} \geq 2000 \\ \frac{16}{Re_{l,v}} & Re_{l,v} < 2000 \end{cases}$$

$$\Phi_l^2 = 1 + (Y^2 - 1) B x_v^{\frac{2-n}{2}} (1 - x_v)^{\frac{2-n}{2}} + x_v^{2-n} \quad \text{where } n = 0.25 \text{ (Blasius)}$$

$$Y = \left(\frac{Y_b}{Y_a}\right)^{1/2}$$

$$Y_a = f_l \frac{2G^2}{D\rho_l}; \quad Y_b = f_v \frac{2G^2}{D\rho_v}$$

$$B = \begin{cases} \frac{55}{\sqrt{G}} & 0 \leq Y < 9.5; G \geq 1900 \\ \frac{2400}{G} & 0 \leq Y < 9.5; 500 \leq G < 1900 \\ 4.8 & 0 \leq Y < 9.5; G < 500 \\ \frac{520}{Y\sqrt{G}} & 9.5 \leq Y < 28; G \leq 600 \\ \frac{21}{Y} & 9.5 \leq Y < 28; G > 600 \\ \frac{15000}{Y^2\sqrt{G}} & Y \geq 28 \end{cases} \quad G \text{ in } \frac{kg}{m^2 s}$$

<p><i>Müller-Steinhagen and Heck [49]</i> (<math>D &gt; 4 \text{ mm}</math>)</p>	$\left(\frac{dp}{dz}\right)_f = G(1 - x_v)^{\frac{1}{3}} + Bx_v^3$ $G = A + 2(B - A)x_v$ $A = \left(\frac{dp}{dz}\right)_l = 4f_l \frac{1}{D} G^2 \left(\frac{1}{2\rho_l}\right)$ $B = \left(\frac{dp}{dz}\right)_l = 4f_v \frac{1}{D} G^2 \left(\frac{1}{2\rho_v}\right)$ $f_{l,v} = \begin{cases} \frac{0.079}{\sqrt[4]{Re_{l,v}}} & Re_{l,v} \geq 2000 \\ \frac{16}{Re_{l,v}} & Re_{l,v} < 2000 \end{cases}$
--	---

385

386

**Tab. 8** – Two-phase pressure drop models for subcooled boiling

Name	Equations
<p><i>Owens-Schrock [50]</i> (<math>3 &lt; D &lt; 4.6 \text{ mm}</math>)</p>	$\frac{\left(\frac{dp}{dz}\right)_{sub}}{\left(\frac{dp}{dz}\right)_{ad}} = \left(0.97 + 0.028e^{6.13\frac{Z_{sub}}{L_{sat}}}\right)$
<p><i>Tong [51]</i> (<math>1.05 &lt; D &lt; 2.44 \text{ mm}</math>)</p>	$\frac{\left(\frac{dp}{dz}\right)_{sub}}{\left(\frac{dp}{dz}\right)_{ad}} = \begin{cases} \left(\frac{Z_{sub}}{L_{sat}}\right)^{1.3} e^{\frac{Z_{sub}}{L_{sat}}+0.4} & \frac{L}{D} = 50 \\ \left(\frac{Z_{sub}}{L_{sat}}\right)^{1.3} e^{\frac{Z_{sub}}{L_{sat}}+1.35} & \frac{L}{D} = 25 \end{cases}$

387

388 The proposed methodology provides encouraging results by using Friedel [12] and Chisholm [14]  
 389 correlations in the subcooled boiling zones. A low agreement has been obtained with the Chawla [48],  
 390 Lockhart-Martinelli [15] and Müller-Steinhagen and Heck [49] correlations. The other two subcooled  
 391 boiling correlations have been also considered (on the right side of the following Tables 9 to 11) to  
 392 compare the results from the present methodology with authoritative and recognized correlations.

393

## 394 7. METHODOLOGY ASSESSMENT

395 The methodology has been applied to the ENEA database from the BO.E.M.I.A. test section. The  
 396 Mean Percentage Error (MPE, Eq. (33)), the Mean Absolute Percentage Error (MAPE, Eq. (34)) and  
 397 the percentage of data within the  $\pm 30\%$  error band have been used to assess the methodology. The  
 398 mean percentage error (MPE) is the computed average of percentage errors by which forecasts of a  
 399 model differ from actual values of the quantity being forecast.

400

$$MPE = \frac{100\%}{N} \sum_{i=1}^n \frac{A-C}{A} \quad (33)$$

401  $A$  is the actual measured value of the quantity being predicted,  $C$  is the calculated value and  $N$  is the  
 402 number of measured values. The formula is useful to understand how far the mean prediction is from  
 403 the data and it has the advantage of neglecting any white noise due to the instrumentation.

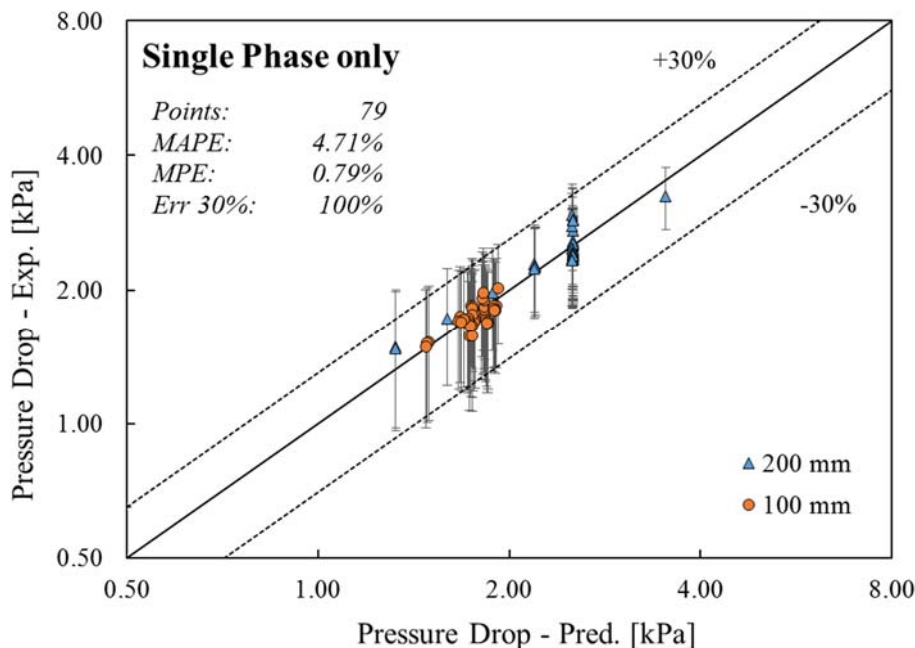
404 The Mean Absolute Percentage Error is:

$$405 \quad MAPE = \frac{100\%}{N} \sum_{i=1}^n \frac{|A-C|}{A} \quad (34)$$

406 MAPE is a quantity used to measure how close predictions are to the eventual outcomes and  
 407 represents the mean error committed for a single forecast.

### 408 7.1 Single-phase transient flow assessment

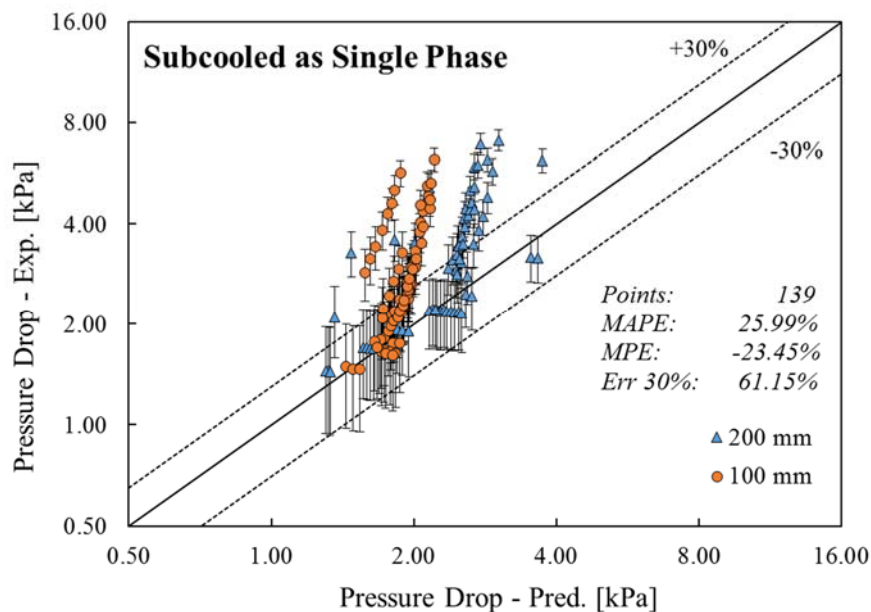
409 Assessing Eq 32 for the pressure drop in transitional regime a good agreement was obtained: 100%  
 410 of the predicted data (79 points) have an error <30%, the MAPE is 4.71% and the MPE 0.79%.  
 411 However, the pressure drop values are quite close to the differential pressure instrument error,  $\pm 510$   
 412 Pa, both in 100 and 200 mm channels, thus the measurements may be affected by an experimental  
 413 error that can strongly affect the model uncertainty. A graphical representation of the results is shown  
 414 in Fig. 9:



415  
 416 **Fig. 9** - Pressure drop, experimental vs predicted in 100 mm and 200 mm tubes.

417  
 418 During the experiment, the average measured single-phase pressure drops were about 2.3 kPa for the  
 419 200-mm tube and 1.7 kPa for the 100 mm. The maximum and minimum values were respectively  
 420 3.95 kPa and 1.49 kPa for the 200-mm tube and 1.76 kPa and 1.44 kPa for the 100 mm tube.

421 Comparing the results with the laminar and turbulent equations (30) and (31), the experimental data  
422 are respectively  $\approx 1/3$  of the Eq. (30) results (laminar) and  $\approx 15$  times compared to the turbulent Eq.  
423 (31). Thanks to its good agreement, once the void fraction is calculated, it can be used in any of the  
424 available pressure drop models before the ONB point. However, it must be noted that the single-phase  
425 pressure drops are quite low (few kPa), thus the instrumentation error is high if compared with the  
426 pressure drops. If all the 139 subcooled points (subcooled boiling only) were considered in single  
427 phase, the predicted pressure drop should be lower than the experimental ones, as shown in Fig. 10,  
428 therefore the single-phase model is not able to predict the pressure drops in the subcooled flow.



429

430

Fig. 10 – Single-Phase pressure drops

431

## 432 7.2 Subcooled flow boiling assessment

433 The assessments for the subcooled flow boiling equations of Tab. 7 are reported in Tab. 9, where all  
434 the points are assessed considering the saturated correlations of Müller-Steinhagen and Heck [49],  
435 Friedel [12], Lockhart-Martinelli [15], Chisholm [14] and Chawla [48] calculated with the vapour  
436 quality and void fraction from Eqs. 15 and 17, and the literature correlations of Owens-Schrock [50]  
437 and Tong [51], specifically developed for subcooled flow boiling. The partial results for the total  
438 pressure drops in the 100 mm and the 200 mm tubes are shown in in Tables 10 and 11, respectively.

439

440

441 **Tab. 9** - Results for subcooled flow boiling points with the present methodology- and literature subcooled  
 442 correlation (all points)

Global (139 points)							
	Present methodology					Subcooled boiling correlations	
Subcooled boiling model	Chisholm [14]	Friedel [12]	Müller-Steinhagen and Heck [49]	Lockhart-Martinelli [15]	Chawla [48]	Owens-Schrock [50]	Tong [51]
MPE	-2.83%	13.48%	22.41%	-32.25%	51.30%	-13.78%	26.09%
MAPE	21.57%	19.02%	24.12%	40.35%	51.32%	32.78%	29.05%
±30%	79.86%	76.26%	67.63%	61.87%	15.11%	65.47%	52.52%

443

444 **Tab. 10** - 100mm results for subcooled flow foiling points with the present methodology and literature  
 445 subcooled correlation

100 mm (76 points)							
	Present methodology					Subcooled boiling correlations	
Subcooled boiling model	Chisholm [14]	Friedel [12]	Müller-Steinhagen and Heck [49]	Lockhart-Martinelli [15]	Chawla [48]	Owens-Schrock [50]	Tong [51]
MPE	-2.44%	14.87%	24.44%	-33.75%	51.38%	-10.46%	29.89%
MAPE	20.82%	17.45%	24.44%	40.42%	51.38%	28.64%	29.89%
±30%	78.95%	78.95%	68.42%	60.53%	15.79%	73.68%	52.63%

446

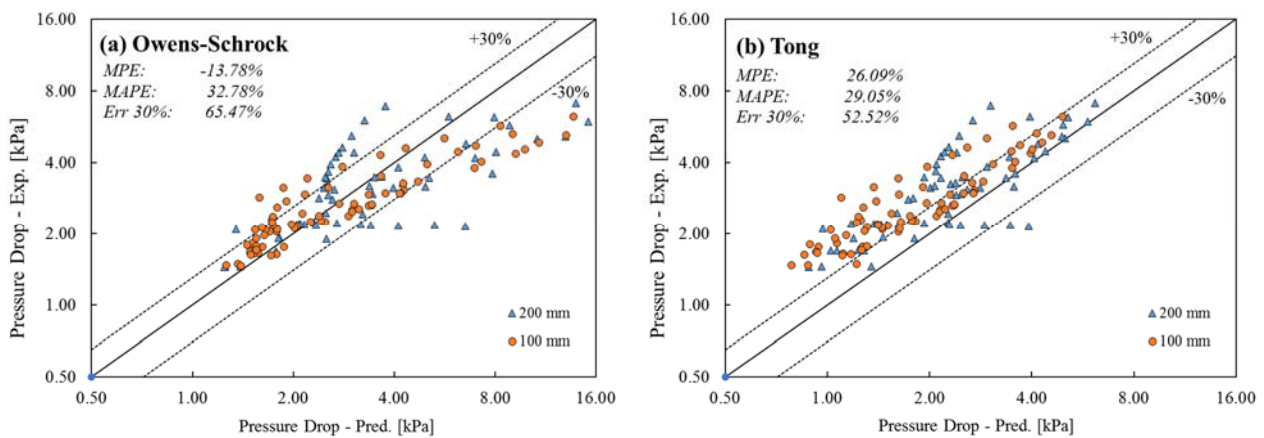
447 **Tab. 11** - 200mm results for subcooled flow foiling points with the present methodology and literature  
 448 subcooled correlation

200 mm (63 points)							
	Present methodology					Subcooled boiling correlations	
Subcooled boiling model	Chisholm [14]	Friedel [12]	Müller-Steinhagen and Heck [49]	Lockhart-Martinelli [15]	Chawla [48]	Owens-Schrock [50]	Tong [51]
MPE	-3.29%	11.81%	19.95%	-30.44%	51.20%	-17.79%	21.50%
MAPE	22.47%	20.90%	23.72%	40.27%	51.25%	37.78%	28.03%
±30%	80.95%	73.02%	66.67%	63.49%	14.29%	55.56%	52.38%

449

450 The subcooled boiling pressure drops trends by using the Owens and Schrock [50] and Tong [51]  
451 correlations are shown in Fig. 11. The same trends for the proposed methodology, adopting different  
452 models, are shown in Fig. 12.

453 The average experimental error is  $\pm 9.5\%$ , ranging between a minimum of  $\pm 3\%$  for high pressure drop  
454 values and  $\pm 35.4\%$  for the lower ones.



455

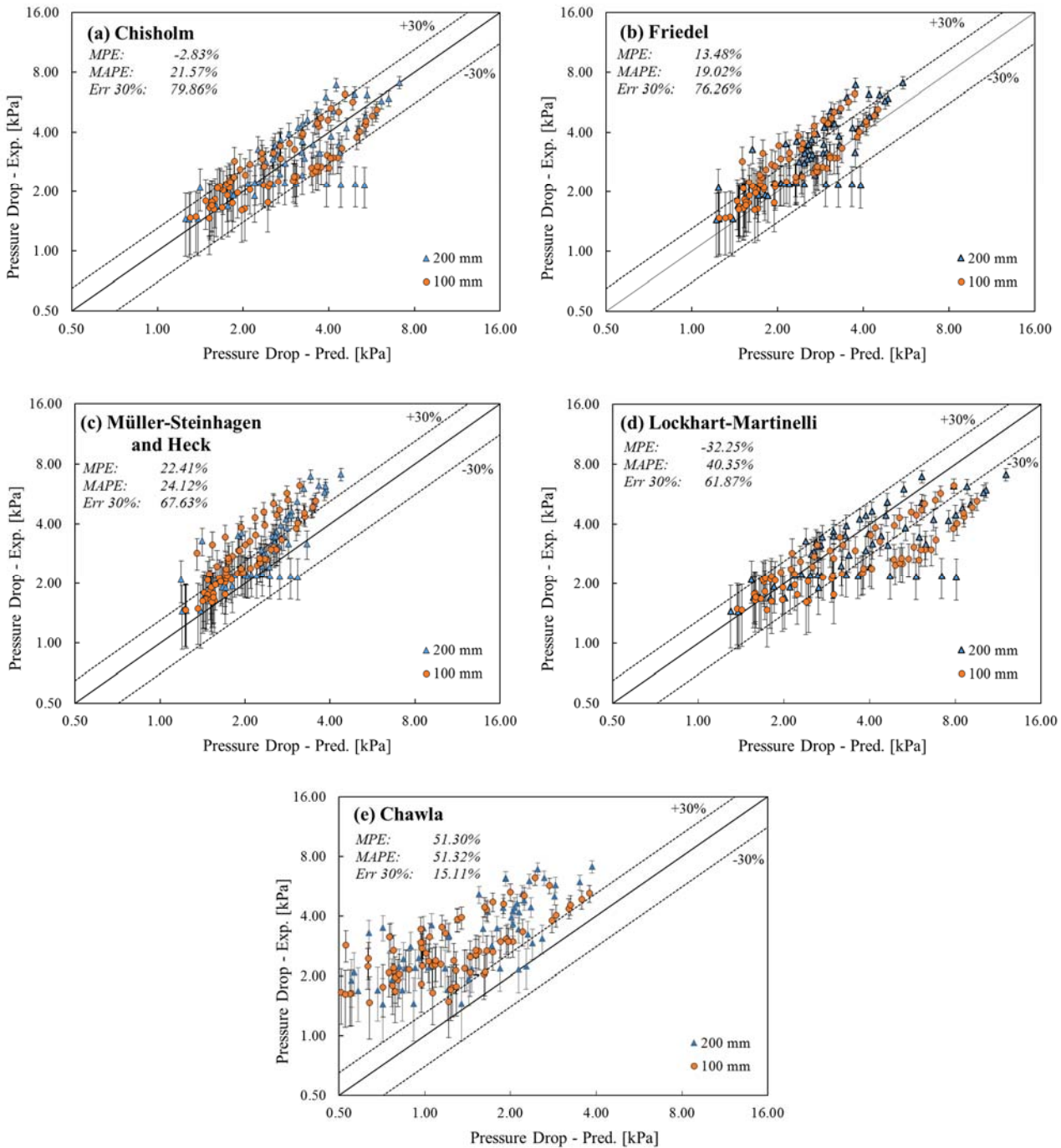
456 **Fig. 11** – Pressure drop prediction for specific subcooled boiling correlations: (a) Owens-Schrock [50], (b)  
457 Tong [51]

458 From Figures 11, 12 and the results summarized in Tab. 9, it is evident how the analysed experimental  
459 data are well predicted when the Chisholm correlation [14] is used. It presents a MAPE of 21.57%  
460 and a MPE of -2.83%, and it is able to predict 79.86% of the points with an error lower than  $\pm 30\%$ .  
461 The best results are for the 100 mm tube, where the Chisholm correlation [14] obtains a MAPE of  
462 20.82% and a MPE of -2.44% with the 78.95% of data within  $\pm 30\%$  error. As it is shown in Fig. 11-  
463 12 and Tab. 8-9-10 the MAPE alone is not always the best instrument to evaluate the prediction  
464 accuracy for a correlation, as it cannot fully describe the quality of a prediction. In fact, any deviation  
465 from the mean values is not evident by this statistical instrument, neither the error sign. The lower  
466 MAPE is for the Friedel correlation [12], but the trends, the MPE and the  $\pm 30\%$  error bands show a  
467 better agreement for Chisholm correlation [14]. The best results obtained from the specific subcooled  
468 boiling correlations come from the Owens-Schrock [50] correlation, where the MPE is -13.78%,  
469 MAPE 32.78 and 65.47% of the data has an error within  $\pm 30\%$ . These correlations present a larger  
470 error than the ones proposed within the methodology and show a wider dispersion, particularly  
471 marked for the 200mm data.



472 The present conclusion is in agreement with Friedel [52] and Tribbe and Müller-Steinhagen [53],  
 473 where the Chisholm [14] correlation was identified as the most suitable one, performing very well in  
 474 calculating pressure drops for  $\mu/\mu_g > 1000$  and with mass velocities greater than  $100 \text{ kg/m}^2\text{s}$ , as in the  
 475 BO.E.MI.A. setup.

476



477

478

479

480 **Fig. 12** –Total pressure drop predictions of subcooled points with the present methodology: (a) Chisholm  
 481 [14], (b) Friedel [12], (c) Müller-Steinhagen and Heck [49], (d) Lockhart-Martinelli [15], and (e) Chawla  
 482 [48].

483 **7.3 Saturated flow boiling assessment**

484 In the saturated zone, it is possible to use another correlation to best fit the experimental data. The  
 485 chosen correlations are the same for subcooled flow boiling: Friedel [12], Lockhart-Martinelli [15],  
 486 Chisholm [14], Chawla [48] and Müller-Steinhagen and Heck [49]. For the assessment, the vapor  
 487 qualities and the actual void fraction are used, following the proposed methodology. As stated in Eq.  
 488 (17), if the equilibrium quality is higher than the vapor quality, the equilibrium quality is used in the  
 489 correlations, but the void fraction is always calculated with Eq. 15 regardless the kind of quality used.  
 490 Tab. 12 summarizes all the results for the total pressure drops (only for the point when a saturated  
 491 zone exists) and Tables 13 and 14 show the results for the 100 mm and 200 mm tubes, respectively.

492 **Tab. 12** - Results for saturated flow boiling points

All points (84 points)					
	Present methodology				
Subcooled boiling model	Lockhart-Martinelli [15]	Chisholm [14]	Chawla [48]	Friedel [12]	Müller-Steinhagen and Heck [49]
MPE	-15.71%	29.55%	44.09%	45.43%	56.77%
MAPE	26.54%	30.25%	44.09%	45.43%	56.77%
±30%	70.24%	44.05%	15.48%	14.29%	0.00%

493

494 **Tab. 13** – 100 mm results for saturated flow boiling points

100 mm (24 points)					
	Present methodology				
Subcooled boiling model	Lockhart-Martinelli [15]	Chisholm [14]	Chawla [48]	Friedel [12]	Müller-Steinhagen and Heck [49]
MPE	-20.35%	24.26%	39.05%	41.28%	53.23%
MAPE	24.67%	25.73%	39.05%	41.28%	53.23%
±30%	75.00%	54.17%	41.67%	20.83%	0.00%

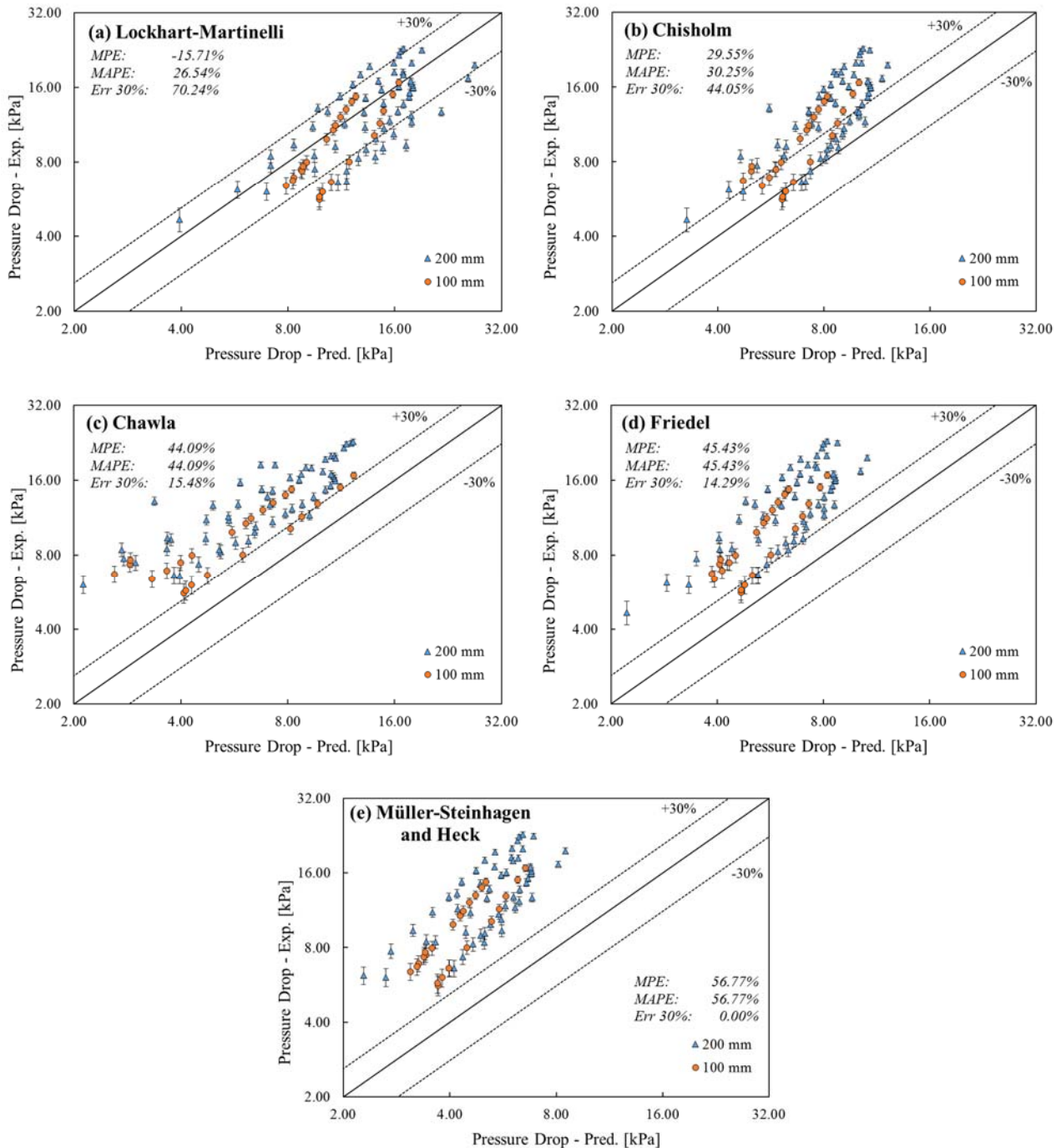
495

496 **Tab. 14** – 200 mm results for saturated flow boiling points

200 mm (60 points)					
	Present methodology				
Subcooled boiling model	Lockhart-Martinelli [15]	Chisholm [14]	Chawla [48]	Friedel [12]	Müller-Steinhagen and Heck [49]
MPE	-13.86%	31.67%	46.11%	47.09%	58.19%
MAPE	27.29%	32.06%	46.11%	47.09%	58.19%
±30%	68.33%	40.00%	5.00%	11.67%	0.00%

497 The saturated flow boiling pressure drops trends by using the proposed methodology, adopting  
498 different models, are shown in Fig. 13.

499



500

501

502

503 **Fig. 13** –Total pressure drop predictions of saturated points: (a) Lockhart-Martinelli [15], (b) Chisholm [14],  
504 (c) Chawla [48], (d) Friedel [12] and (e) Müller-Steinhagen and Heck [49].

505

506 Fig. 13 and the results summarized in Tab. 12 show how the predictions underestimate the  
507 experimental data in most of the correlations, suggesting the use of a higher void fraction in the  
508 channel. However, the Lockhart-Martinelli [15] correlation, that overestimated the data in subcooled  
509 flow boiling region, have a low error in saturated conditions; its MPE is -15,71%, the MAPE 26,54%  
510 and 70,24% of the data have an error lower than  $\pm 30\%$ . The second-best correlation is the Chisholm  
511 [14] one, with and MPE of 29,55% a MAPE of 30,25% and 44,05% of the data with an error lower  
512 than  $\pm 30\%$ . This suggests the use of the Lockhart-Martinelli [15] correlation to obtain a good  
513 prediction or the Chisholm [14] correlation in first approximation, as the latter provides good  
514 predictions in the subcooled boiling.

515

#### 516 7.4 Methodology assessment

517 Merging the best results obtained from the single phase transient pressure drop obtained from Eq.  
518 (26), from Chisholm model [14] for the subcooled flow boiling and the best saturated flow boiling  
519 results, obtained with the Lockhart-Martinelli [15] model (both calculated with the void fraction of  
520 Eq. 15 and vapor quality of Eq. 17), the performance summarized in Tab. 15 has been obtained. The  
521 100 mm tube get the best results with an MPE of -5.25% a MAPE of 16.60% and 84.40% of the data  
522 with an error lower than  $\pm 30\%$ . The global results, for both 100 mm and 200 mm channels, are quite  
523 good and near to the 100 mm tube results; the MPE is -5.88%, MAPE is 18.54% and 82.45% of the  
524 data has an error lower than  $\pm 30\%$ .

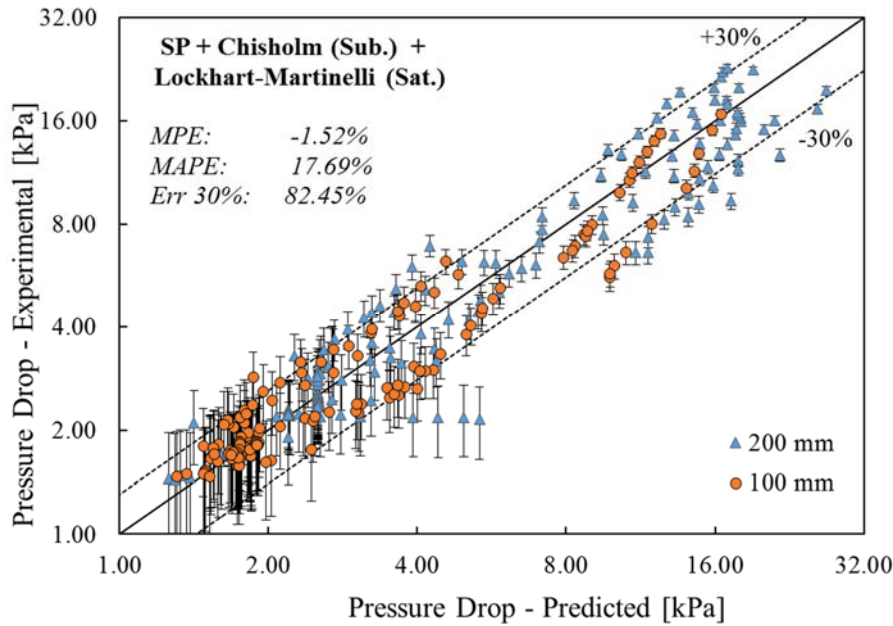
525

**Tab. 15** – Results for the proposed methodology

	100 mm	200 mm	All points
Points	141	161	302
MPE	-5.25%	-6.43%	-5.88%
MAPE	16.60%	20.24%	18.54%
$\pm 30\%$	84.40%	80.75%	82.45%

526

527 The total pressure drops trends obtained by using the proposed methodology, and adopting the model  
528 for single-phase in transition flow, Chisholm [14] for subcooled boiling and Lockhart-Martinelli [15]  
529 for saturated boiling, as described above, are shown in Fig. 14. It is also possible to use of Friedel  
530 [12] correlation for the subcooled zone, as the choice of using Chisholm [14] is due to the lower MPE  
531 joined to the largest number of data points predicted within  $\pm 30\%$ .



532

533

**Fig. 14** –Total pressure drop predictions for the whole ENEA data base.

534 The average pressure drop (experimental) was 3.69 kPa in the 100 mm channel, the minimum value  
535 was 1.47 kPa and the maximum 16.74 kPa; for the 200 mm channel, the three values were: 6.87 kPa,  
536 1.44 kPa and 22.87 kPa, respectively.

537

## 538 8. CONCLUSIONS

539 The present study deals with a methodology to evaluate pressure drops in small tubes, using an ENEA  
540 data base for a mini tube with a diameter of 1 mm and two different lengths (100 and 200 mm,  
541 respectively), and its validation when the flow is in transition conditions. The main features of the  
542 methodology are the capability to be used also in transition flow and in the use of non-equilibrium  
543 vapor quality instead of the equilibrium thermodynamic quality.

544 The calculation method includes single phase, subcooled and saturated boiling zones, identifying their  
545 boundaries. Employing a third order interpolation curve, the pressure drop for subcooled liquid in  
546 transition flow can be calculated. The methodology is based on the work of Delhayé et al. [28]. The  
547 model considers the fluid properties, the energy, mass and momentum conservation to predict the  
548 ONB, OSV points and a hyperbolic function is adopted to calculate the non-equilibrium vapor quality  
549 in the subcooled boiling region.

550 The vapor quality and void fraction are used in well-known pressure drop models, such as: Friedel  
551 [12], Chisholm [14], Chawla [48], Lockhart-Martinelli [15] and Müller-Steinhagen and Heck [49].

552 The results have been also compared with the correlations from Owens-Schrock [50] and Tong [51],  
 553 specific for subcooled flow boiling.

554 The best agreement with the ENEA experimental data has been obtained using the transition model  
 555 for the single-phase flow region, the Chisholm [14] model for the subcooled flow boiling region and  
 556 the Lockhart-Martinelli [15] for the saturated flow boiling region. The resulting MAPE is of 18,54%,  
 557 a MPE of -5,88% and 82,45% of the predicted points with an error lower than 30%, in a data base of  
 558 302 points. The results are very encouraging because none of the employed correlation was developed  
 559 specifically for the ENEA database or adapted on it, with the only exception of a small reduction of  
 560 the  $Pr$  number exponent (0.95 instead of 1) in the Frost and Dzakowic [31] correlation in the ONB  
 561 prediction. Further pressure drop correlations that can be used with the proposed methodology are  
 562 available in the literature. Moreover, all the methodology steps have been checked with an  
 563 “applicability model”, proposed by Delhay et al. [28], to assure the compatibility with the fluid and  
 564 ranges used in the experimental facility.

565

#### 566 **Appendix –Summary of the methodology**

567 The total pressure drop in the tube can be calculated as:

$$568 \int_0^{Z_t} \left( \frac{dp}{dz} \right)_T = \int_0^{Z_{ONB}} \left( \frac{dp}{dz} \right)_{f,SP} + \int_{Z_{ONB}}^{Z_t} \left( \frac{dp}{dz} \right)_{f,TP} + \int_{Z_{ONB}}^{Z_t} \left( \frac{dp}{dz} \right)_{acc} + \int_0^{Z_t} \left( \frac{dp}{dz} \right)_g \quad (A1)$$

569 The gravitational contribution is neglected if the tube is horizontal ( $\gamma = 0$ ):

$$570 \int_0^{Z_t} \left( \frac{dp}{dz} \right)_g = \int_0^{Z_t} \rho(Z)g \cdot dZ \sin\gamma \quad (A2)$$

571 The acceleration contribution is calculated in two-phase only, neglecting the density variation in the  
 572 single-phase zone:

$$573 \int_{Z_{ONB}}^{Z_t} \left( \frac{dp}{dz} \right)_{acc} = \int_{Z_{ONB}}^{Z_t} \frac{G^2}{\rho_l} \left( \frac{\rho_l x_v^2}{\rho_v \alpha} + \left( \frac{(1-x_v)^2}{(1-\alpha)} - 1 \right) \right) \cdot dZ \quad (A3)$$

574

#### 575 *A: Regions boundaries (ONB and Saturation points, Sect. 4)*

576 The single-phase region ends at the onset of nucleation boiling point, which can be identified as:

$$577 Z_{ONB} = \frac{Gc_{pl}D}{4} \cdot \left[ \frac{((T_{sat}-T_{lin})+(\Delta T_{sat})_{ONB})}{Q} - \frac{1}{h_{l,conv}} \right] \quad (A4)$$

578 where the wall superheating at ONB is obtained from the modified Frost and Dzakowic correlation  
 579 [31]:

$$580 \quad (\Delta T_{sat})_{ONB} = \left( \frac{8\sigma Q T_{sat}}{k_{l,sat} H_{lv,sat} \rho_g} \right)^{0.5} Pr_l^{0.95} \quad (A5)$$

581 The saturation length is obtained by a simple energy balance:

$$582 \quad L_{sat} = \frac{G \cdot D}{4 \cdot Q} \int_{T_{in}}^{T_{sat}} c_{pl} \cdot dT \quad (A6)$$

583 from which the saturation point is identified as  $Z_{sat} = L_{sat}$ , if  $L_{sat}$  is less or equal than the tube length.

584

585 *B: Single-phase pressure drops (Sect. 6.1)*

586 The pressure drops in single phase are evaluated with:

$$587 \quad \int_0^{Z_{ONB}} \left( \frac{dp}{dz} \right)_{f,SP} = \int_0^{Z_{ONB}} f \cdot \frac{G^2}{2D\rho_l} \cdot dZ \quad (A7)$$

588 where the friction factor  $f$  is calculated through Eqs. (28) to (31) to consider the transition flow regime.

589

590 *C: Void fraction and actual vapour quality (Sect. 5)*

591 To calculate the acceleration and the two-phase frictional contributions, the evaluation of the void  
 592 fraction is needed. It is calculated by the Lahey and Moody model [34]:

$$593 \quad \alpha = \frac{x_v \cdot \rho_l \cdot G}{C_0 (x_v \cdot \rho_l + (1-x_v) \cdot \rho_v) G + V_g \cdot \rho_l \cdot \rho_v} \quad (A8)$$

594 The distribution parameter  $C_0$  is evaluated by Eq. (22) and the weighted drift velocity by Eq. (25).

595 In the partial developed (between the ONB and the OSV points) and in the fully developed (from  
 596 OSV to saturation) boiling regions, the non-equilibrium vapor quality is assumed, following Delaye  
 597 et al. [28]:

$$598 \quad x_v(Z) = 0.01\xi \left\{ x_{eq}(Z) - x_{eq}(Z_{ONB}) \left[ \tanh \left( \left( \frac{x_{eq}(Z)}{x_{eq}(Z_{ONB})} \right) - 1 \right) + 1 \right] \right\} \quad (A9)$$

599 where the equilibrium quality (negative in this zones) is evaluated by Eq. (16). The parameter  $\xi$  is  
 600 evaluated imposing the continuity at  $Z_{OSV}$  (Eq. (13)) with the quality calculated by Eqs. (18) to (21).

601

602 *D: Two-phase pressure drops (Sect. 6.2)*

603 The Chisolm model [14] provided the best agreement with ENEA data in the subcooled flow boiling  
604 region. It can be used in the saturated region also, but the best agreement in saturation has been  
605 provided by the Lockart-Martinelli model [15] (see Table 6 for the models' details).

606

607

## 608 **Nomenclature**

609	A	actual value
610	Bd	Bond number
611	Bo	Boiling number
612	$C_0$	distribution parameter
613	$c_p$	specific heat, J/kgK
614	C	parameter in Lockhart-Martinelli correlation; calculated value
615	D	tube diameter, m
616	Fr	Froude number
617	f	Fanning friction factor
618	G	mass flux, kg/m <sup>2</sup> s
619	g	gravitational acceleration, 9.806 m/s <sup>2</sup>
620	$H_{lv}$	latent heat, J/kg
621	H	enthalpy, J/kg
622	h	heat transfer coefficient, W/m <sup>2</sup> K
623	k	thermal conductivity, W/mK
624	L	tube length, m
625	N	number of data points
626	n	coefficient
627	Pr	Prandtl number
628	p	pressure, Pa
629	Q	heat flux, W/m <sup>2</sup>
630	Re	Reynolds number
631	S	suppression factor
632	T	temperature, K
633	$V_g$	weighted drift velocity, m/s
634	We	Weber number
635	$X_{tt}$	Lockhart-Martinelli parameter
636	x	quality
637	Y	Chisholm correlation coefficient
638	Z	axial coordinate (stream-wise)

## 639 *Greek symbols*

640	$\alpha$	void fraction
641	$\Gamma$	volumetric flow rate, m <sup>3</sup> /s
642	$\mu$	dynamic viscosity, Pa s
643	$\xi$	convergence parameter
644	$\rho$	density, kg/m <sup>3</sup>
645	$\sigma$	surface tension, N/m
646	$\varphi$	two-phases multiplier



647 *Subscripts*

648	conv	convective
649	eq	equilibrium
650	F	friction
651	H	heated
652	in	inlet
653	lam	laminar
654	lo	liquid only
655	l	liquid
656	out	outlet
657	$\Delta P$	pressure drop
658	sat	saturated
659	sub	subcooled
660	SP	single-phase
661	TP	two-phase
662	turb	turbulent
663	t	total
664	v	vapor
665	vo	vapor only
666	w	wall
667	z	axial

668

669 *Acronyms*

670	MAPE	Mean Average Percentage Error
671	MPE	Mean Percentage Error
672	ONB	Onset Nucleate Boiling point
673	OSV	Onset Significant Void point
674	PDB	Partial Developed Boiling region
675	FDB	Full Developed Boiling region

676

677 **References**

- 678 [1] S.M. You, T.M. Simon, A. Bar-Cohen, Experiments on Nucleate Boiling Heat Transfer with a Highly-  
679 Wetting Dielectric Fluid: Effects of Pressure, Subcooling and Dissolved Gas Content, *Cryogenic and*  
680 *Immersion Cooling of Optics and Electronic Equipment*, HTD-Vol. 131 (1990), 45–52.
- 681 [2] M. Cairra, G. Caruso, A. Naviglio, A correlation to predict CHF in subcooled flow boiling, *Int Comm.*  
682 *in Heat and Mass Transfer*, Vol. 22 (1995), 35-45.
- 683 [3] A.M. Jacobi, S.S. Mehendale, R.K. Shah, Fluid flow and heat transfer at micro- and meso-scales with  
684 application to heat exchanger design, *Applied mechanics review*, 53 (2000), 175–193.
- 685 [4] G. Kandlikar, Scale effect on flow boiling heat transfer in microchannels: a fundamental perspective.  
686 *International journal of thermal sciences*, 49 (2010), 1073–1085.
- 687 [5] P. Cheng and H.Y. Wu. Mesoscale and microscale phase-change heat transfer. *Advances in heat*  
688 *transfer* (2006), 39:469–573.
- 689 [6] M.B. Ould Didi, N. Kattan, J.R. Thome, Prediction of Two-Phase Pressure Gradients of Refrigerants  
690 in Horizontal Tubes, *International Journal of Refrigeration* 25 (2002), 935-947.
- 691 [7] H. Muller-Steinhagen, K. Heck. A simple friction pressure drop correlation for two-phase flow in  
692 pipes. *Chem. Eng. Processing* 20 (1986), 297–308.
- 693 [8] R. Gronnerud, Investigation of liquid hold up, flow resistance and heat transfer in circulation type  
694 evaporators, Part IV: two phase flow resistance in boiling refrigerants, Annex 1972-1, *Bull Del'Inst.*  
695 *Du Froid* (1972).

- 696 [9] G. Ribatski, L. Wojtan, J.R. Thome, An analysis of experimental data and prediction methods for two-  
697 phase frictional pressure drop and flow boiling heat transfer in micro-scale channels. *Experimental*  
698 *thermal and fluid science* 31-1 (2006), 1–19.
- 699 [10] M. Zhang, R.L. Webb, Correlation of two-phase friction for refrigerants in small-diameter tubes, *Exp.*  
700 *Thermal Fluid Sci.* 25 (2001) 131–139.
- 701 [11] K. Kuwahara, S. Koyama, K. Kazari, Experimental study of flow boiling of HFC134a in a multi-port  
702 extruded tube, ICMM2004, Rochester, New York, USA (2004).
- 703 [12] L. Friedel, Improved friction pressure drop correlations for horizontal and vertical two-phase pipe  
704 flow, European Two-Phase Flow Group Meeting, Ispra, Italy, Paper E2 (1979).
- 705 [13] G.M. Lazarek, S.H. Black, Evaporative heat transfer, pressure drop and critical heat flux in a small  
706 vertical tube with R-113, *Int. J. Heat Mass Transfer* 25 (1982) 945–960.
- 707 [14] D. Chisholm, A theoretical basis for the Lockhart-Martinelli correlation for two-phase flow, *Int. J.*  
708 *Heat Mass Transf.* 10-18 (1967), 1767–1778.
- 709 [15] R.W. Lockhart, R.C. Martinelli, Proposed correlation of data for isothermal two-phase two-component  
710 flow in pipes, *Chem. Eng. Progr.* 45-1 (1949), 39–48.
- 711 [16] W. Qu, I. Mudawar, Measurement and prediction of pressure drop in two-phase micro-channel heat  
712 sinks, *int. J. Heat mass transf.* 46 (2003), 2737–2753.
- 713 [17] J. Lee, I. Mudawar, Two-phase flow in high-heat-flux micro-channel heat sink for refrigeration cooling  
714 applications: part I – pressure drop characteristics, *Int. J. Heat mass transf.* 48 (2005), 928–940.
- 715 [18] P.S. Lee, S.V. Garimella, Saturated flow boiling heat transfer and pressure drop in silicon  
716 microchannel arrays, *Int. J. Heat mass transf.* 51 (2008), 789–806.
- 717 [19] K. Mishima, T. Hibiki, Some characteristics of air–water two-phase flow in small diameter vertical  
718 tubes, *Int. J. Multiphase Flow* 22 (1996) 703–712
- 719 [20] M.B. Bowers, I. Mudawar, Two-phase electronic cooling using mini-channel and micro-channel heat  
720 sinks: part 1 – Design Criteria and Heat Diffusion Constraints, *J. Electron. Packag.* 116 (1994), 290–  
721 297.
- 722 [21] M.B. Bowers, I. Mudawar, Two-phase electronic cooling using mini-channel and micro-channel heat  
723 sinks: Part 2— flow rate and pressure drop constraints, *J. Electron. Packaging* 116 (1994) 298–305.
- 724 [22] S.G. Kandlikar, M.E. Steinke, S. Tian, L.A. Campbell, High-speed photographic observation of flow  
725 boiling of water in parallel mini-channels, *Proceedings of 35th National Heat Transfer Conference,*  
726 *ASME, Anaheim, CA (2001), 675–684.*
- 727 [23] G. Hetsroni, A. Mosyak, Z. Segal, G. Ziskind, A uniform temperature heat sink for cooling of  
728 electronic devices, *Int. J. Heat Mass Transfer* 45 (2002), 3275– 3286.
- 729 [24] T.N Tran, M.-C Chyu, M.W Wambsganss, D.M France, Two-phase pressure drop of refrigerants  
730 during flow boiling in small channels: an experimental investigation and correlation development,  
731 *International Journal of Multiphase Flow* 26-11 (2000), Pages 1739-1754.
- 732 [25] S.M. Kim, I. Mudawar, Universal approach to predicting two-phase frictional pressure drop for  
733 mini/micro-channel saturated flow boiling, *Int. J. Heat Mass Transf.* 58 (1–2) (2013), 718–734.
- 734 [26] L. Saraceno, G.P. Celata, M. Furrer, A. Mariani, G. Zummo, Flow boiling heat transfer of refrigerant  
735 FC-72 in microchannels, *International Journal of Thermal Sciences* 53 (2012), 35-41.
- 736 [27] L. Gugliemetti, G. Caruso, L. Saraceno, G. Zummo, G.P. Celata, Saturated flow boiling of FC-72 in  
737 1 mm diameter tube, *International Communications in Heat and Mass Transfer* 75 (2016), 115-123.
- 738 [28] J. M. Delhaye, F. Maugin, J.M. Ochterbeck, Void fraction predictions in forced convective subcooled  
739 boiling of water between 10 and 18 MPa, *International Journal of Heat and Mass Transfer* 47 19-20  
740 (2004), 4415-4425.
- 741 [29] J.M. Kay, R.M. Nedderman, *An Introduction to Fluid Mechanics and Heat Transfer*, third ed.,  
742 Cambridge University Press, Cambridge (1974), 134-144.
- 743 [30] G. R Celata, M. Cumo, and T. Setaro Hysteresis phenomena in subcooled flow boiling of well-wetting  
744 fluids, *Experimental Heat Transfer* 5-4 (1992), 54-75.
- 745 [31] W. Frost, G.S. Dzakowic, An extension of the method of predictive incipient boiling on commercially  
746 finished surfaces, *ASME-AICHe Heat Transfer Conference, Seattle (1967).*

- 747 [32] P. Saha, N. Zuber, Point of net vapor generation and vapor void fraction in subcooled boiling, Fifth  
748 Int. Heat Transfer Conf., Tokyo, Paper B4.7 (1974).
- 749 [33] N. Zuber, J.A. Findlay, Average volumetric concentration in two phase flow systems, J. Heat Transfer  
750 87 (1965), 453–467.
- 751 [34] R.T. Lahey Jr., F.J. Moody, The Thermal Hydraulics of a Boiling Water Nuclear Reactor, American  
752 Nuclear Society, LaGrange Park (1977).
- 753 [35] S. Levy, Forced convection subcooled boiling-prediction of vapor volumetric fraction, Int. J. Heat  
754 Mass Transfer 19 (1967), 99–113.
- 755 [36] P. Griffith, J.A. Clark, W.M. Rohsenow, Void volumes in subcooled boiling systems, ASME Paper  
756 58-T-19, New York (1958).
- 757 [37] H. Nabizadeh, Übertragungsgesetze für den Dampfvolumenteil zwischen Freon und Wasser,  
758 Kolloquium über Ähnlichkeitsgesetze in Zweiphasenströmungen, Institut für Verfahrenstechnik der  
759 T.U. Hannover (1980).
- 760 [38] G.E. Dix, Vapor Void Fractions for Forced Convection with Subcooled Boiling at Low Flow Rate,  
761 Ph.D. Thesis, University of California (1971).
- 762 [39] S.M. Kim, I. Mudawar, Consolidated method to predicting pressure drop and heat transfer coefficient  
763 for both subcooled and saturated flow boiling in micro-channel heat sinks, International Journal of  
764 Heat and Mass Transfer, Volume 55 13–14 (2012), 3720-3731.
- 765 [40] M. Margulis, E. Shwageraus, Extending Two-Phase Capabilities of Thermal-Hydraulic Module in  
766 BGCore, Reactor Physics and Technology II (In memory of Dr. Uri Mintzer) (2014).
- 767 [41] V.S. Osmachkin, V. Borisov, Pressure drop and heat transfer for flow boiling water in vertical rod  
768 bundles, Paper B4.9, IVth International Heat Transfer Conference Paris-Versailles (1970).
- 769 [42] W.R. Brownlie, Reexamination of Nikuradse roughness data, Journal of the Hydraulics Division-  
770 ASCE, 107-1 (1981), 115-119.
- 771 [43] N.S. Cheng, Y.M. Chiew, Modified logarithmic law for velocity distribution subjected to upward  
772 seepage, Journal of Hydraulic Engineering, ASCE 124-12 (1998), 1235-1241.
- 773 [44] P.M. Ligrani, R.J. Moffat, Structure of Transitionally Rough and Fully Rough Turbulent Boundary-  
774 Layers, Journal of Fluid Mechanics 162 (1986), 69-98.
- 775 [45] M. S. Yalin, A.M.A.F. Da Silva, Fluvial processes, IAHR, Delft, Netherlands (2001).
- 776 [46] N.S. Cheng, Formulas for friction factor in transitional regions, Journal of Hydraulic Engineering,  
777 ASCE 134-9 (2008), 1357-1362.
- 778 [47] C.F. Colebrook, Turbulent flow in pipes with particular reference to the transition region between the  
779 smooth and rough pipe laws, Proceedings of the Institution of Civil Engineers, 12 (1939), 393-422.
- 780 [48] J.M. Chawla, Wärmeübergang und Druckabfall in Waagerechten Rohren bei der Strömung von  
781 Verdampfenden Kältemitteln, VDI-Verlag (1967).
- 782 [49] H. Müller-Steinhagen, K. Heck, A simple friction pressure drop correlation for two-phase flow in  
783 pipes, Chem. Process Engineering, Vol. 20 (1986), pp. 297-308.
- 784 [50] W.L. Owens, V.E. Schrock, Local pressure gradients for subcooled boiling of water in vertical tubes,  
785 ASME Paper No. 60-WA-249 (1960).
- 786 [51] W. Tong, A.E. Bergles, M.K. Jensen, Pressure drop with highly subcooled flow boiling in small-  
787 diameter tubes, Exp. Therm. Fluid Sci. 15 (1997), 202–212.
- 788 [52] L. Friedel, Momentum exchange and pressure drop in two-phase, NATO Advanced Study Institute,  
789 Istanbul, Turkey (1975) (Also Appendix B, P. Whalley “Boiling Condensation and Gas-Liquid Flow”,  
790 Oxford, 1987).
- 791 [53] C. Tribbe, H. Müller-Steinhagen. An Evaluation of the Performance of Phenomenological models for  
792 Predicting Pressure Gradient during Gas-Liquid Flow in Horizontal Pipelines, Int. J. Multiphase Flow  
793 26 (2000), 1019-1036.